

# Nuclear Fusion and Plasma Physics

## Lecture 11

**Ambrogio Fasoli**

Swiss Plasma Center

Ecole Polytechnique Fédérale de Lausanne

# Applied superconductivity for fusion

## Layout of the lecture

The need for superconducting magnets

Superconductivity – generalities

Requirements and challenges

Fusion devices with superconducting coils

ITER, DEMO and beyond

*Presentation by Nikolay Bykovskiy on his project in Applied Superconductivity for fusion*

# The need for superconducting magnets

Plasma confinement needs high magnetic fields over large volumes

Increasing  $B$  is key for performance of magnetic fusion reactors

$n\tau_E T$  scales with  $B^\alpha$ , where  $\alpha \geq 2$

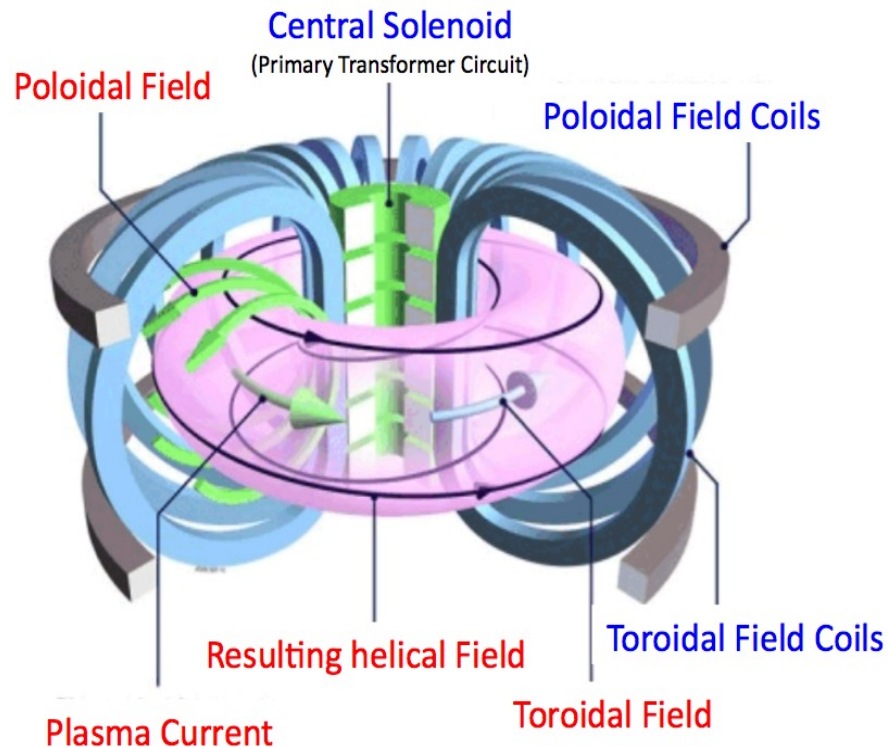
Copper coils can generate large fields, but not in steady-state

Current density in steady-state  $\leq 10 \text{ A/mm}^2$

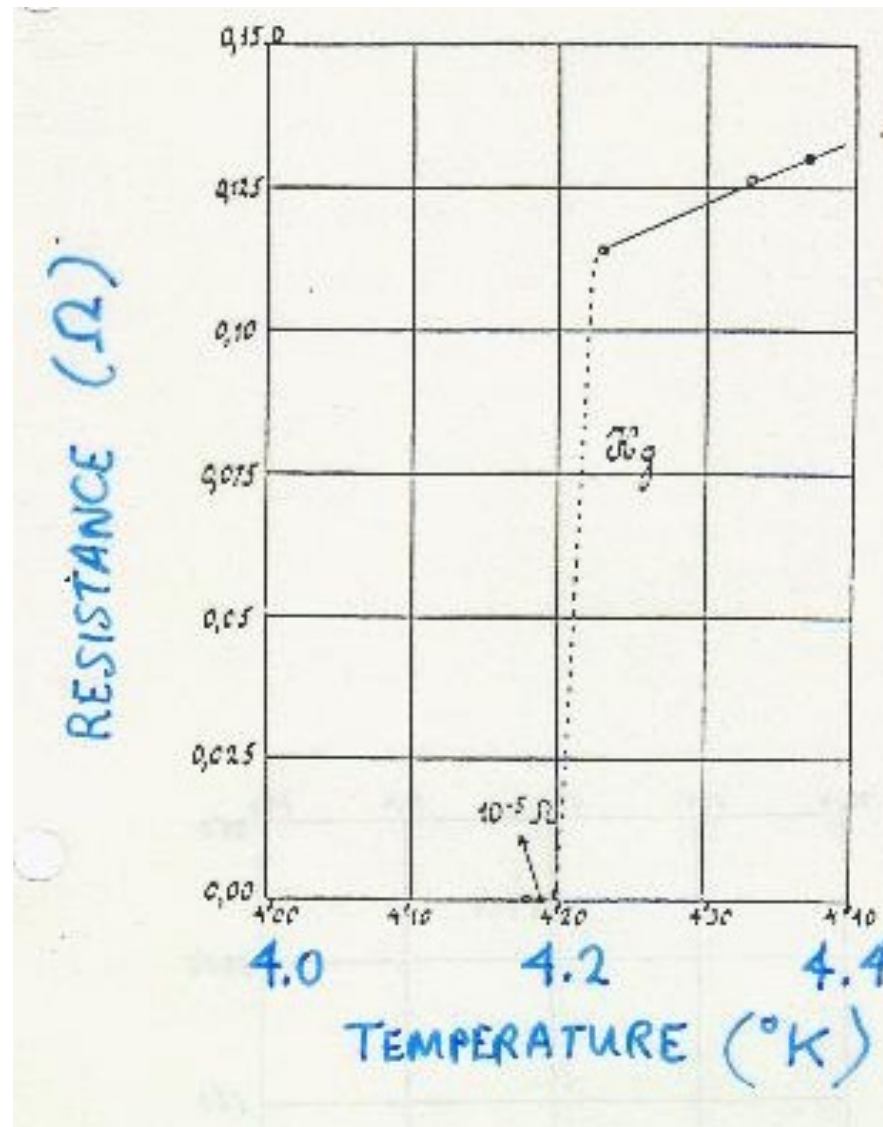
For steady-state, superconductors are necessary

Current density in steady-state  $\leq 1000 \text{ A/mm}^2$

Low dissipation in coils, low recirculating power



# The discovery of superconductivity

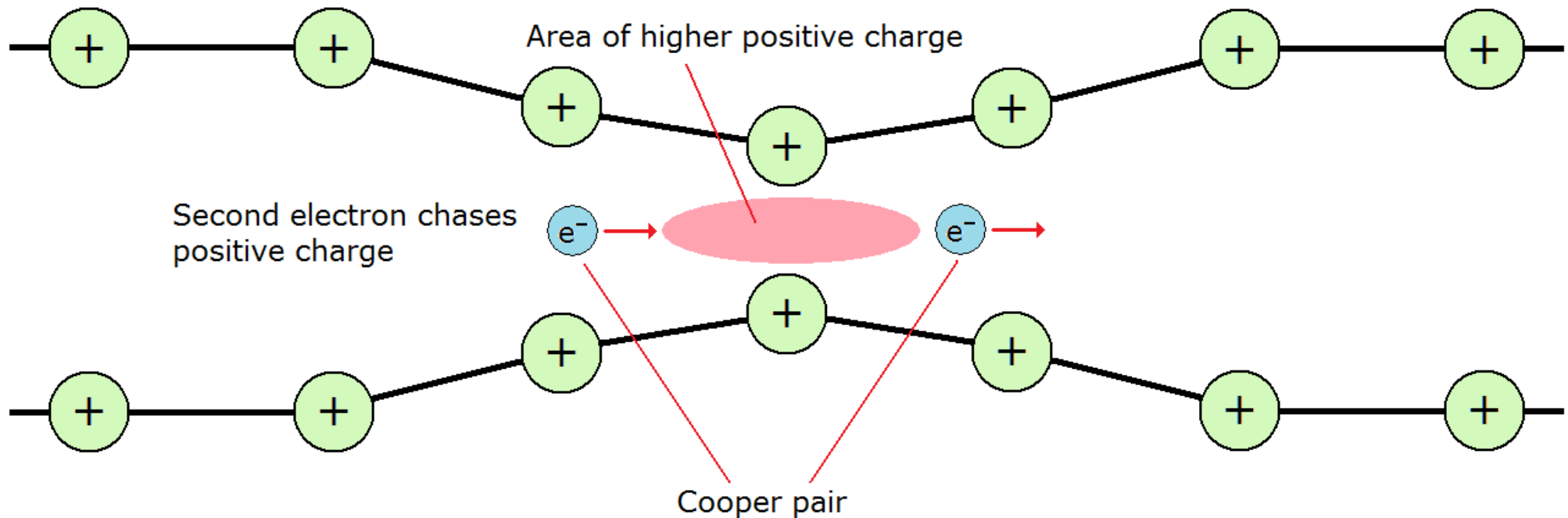


In 1911 Kamerlingh Onnes observed that the electrical resistance of a sample of mercury becomes exactly 0 when the sample is brought at the temperature of liquid Helium



# EPFL Superconductivity – simple interpretation

BCS theory (1972): below a critical temperature  $T_c$ , an effective attraction between pairs of electrons (Cooper pairs) via the lattice promotes a condensed state in which the phases of all the individual wave functions are locked together



<https://dc.edu.au/wp-content/uploads/cooper-pair-phonon.png>

Contrary to the unpaired electrons with spin  $\frac{1}{2}$  (fermions), the Cooper pairs have integer spin, i.e. are bosons and can be in the same quantum state, moving resistance-less through lattice

# EPFL Superconductivity – simple interpretation

<https://www.youtube.com/watch?v=O6sukls0ozk>

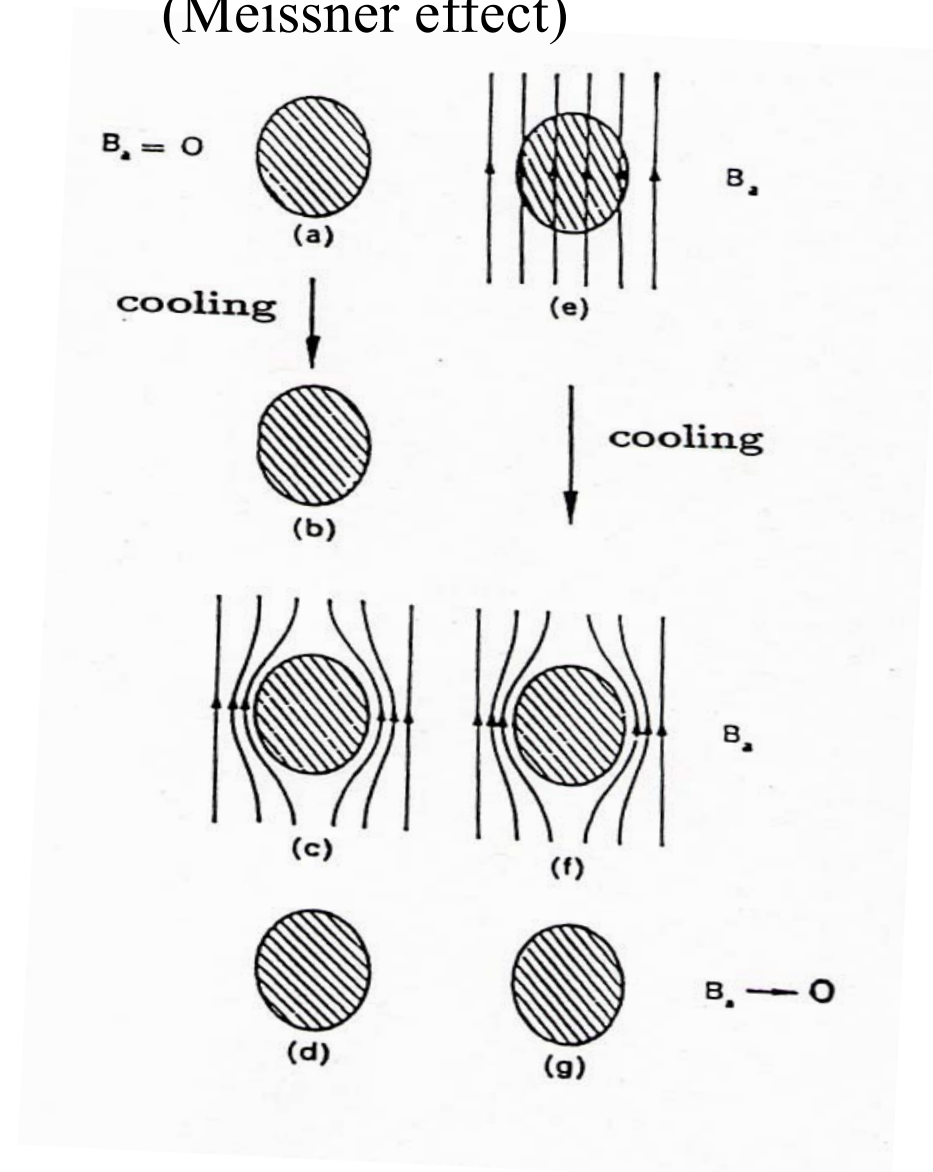
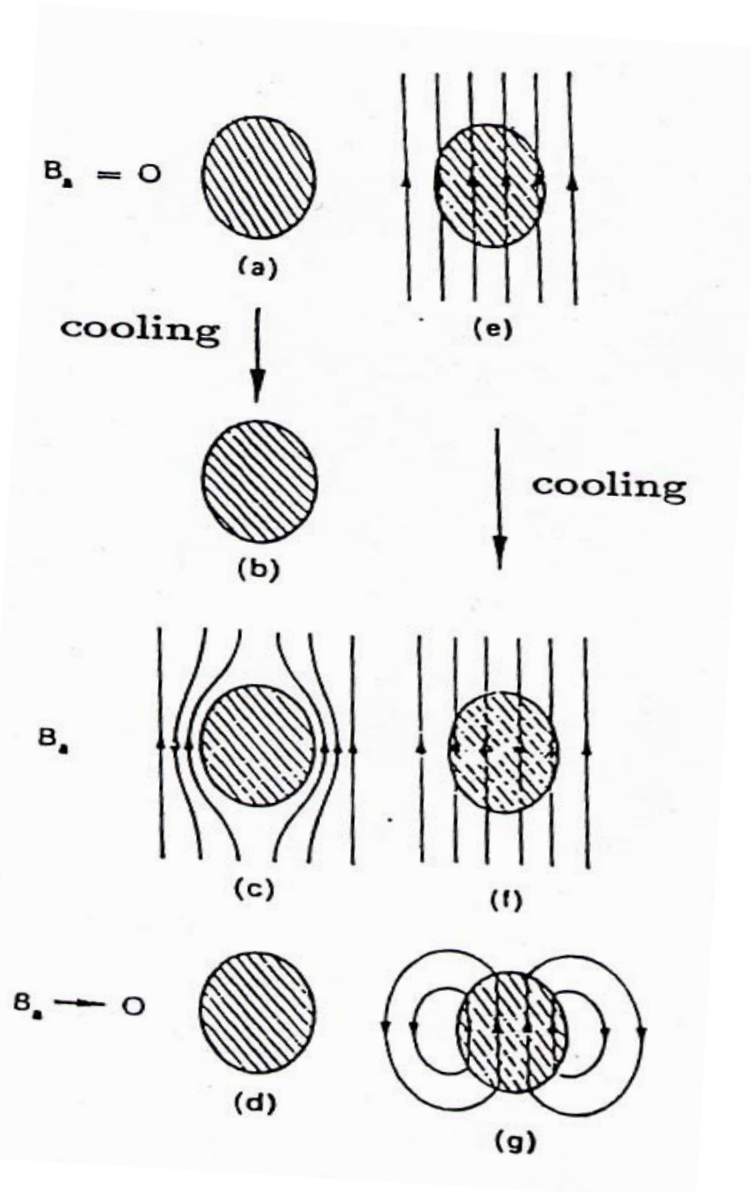


# Superconductors vs. perfect conductors

Perfect conductors  $R=0, dB/dt=0$

Superconductors  $R=0, B=0$

(Meissner effect)



# EPFL Magnetization and Type I vs. Type II SC's

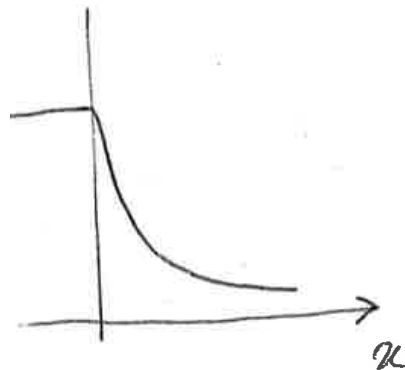
Magnetic flux is excluded from the bulk of superconductors by screening currents flowing at the surface, within the London penetration depth,  $\lambda$

In the superconductor (London theory, 1935):

$$\nabla^2 B = \frac{B}{\lambda^2}$$

$$\lambda^2 = \frac{m_e}{2e^2 \mu_0 n_C} \quad n_C = \text{density of sc carriers}$$

At the boundary  $B = B_0 e^{-\frac{x}{\lambda}}$



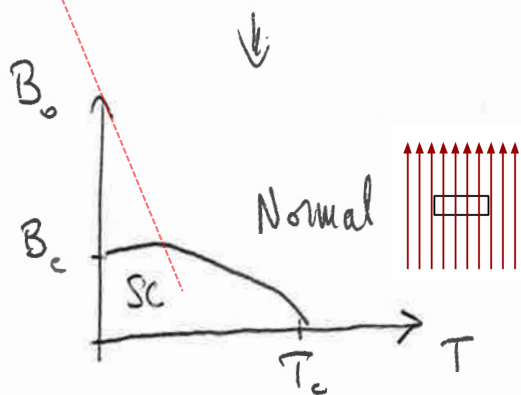
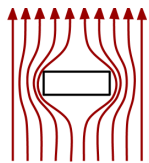
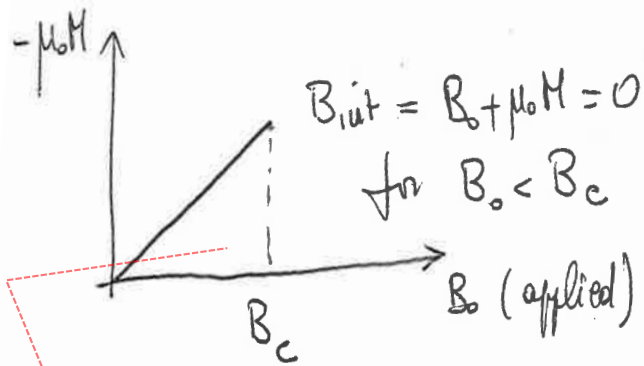


# EPFL Magnetization and Type I vs. Type II SC's

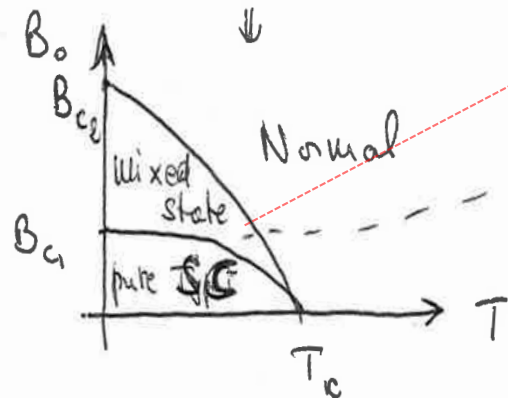
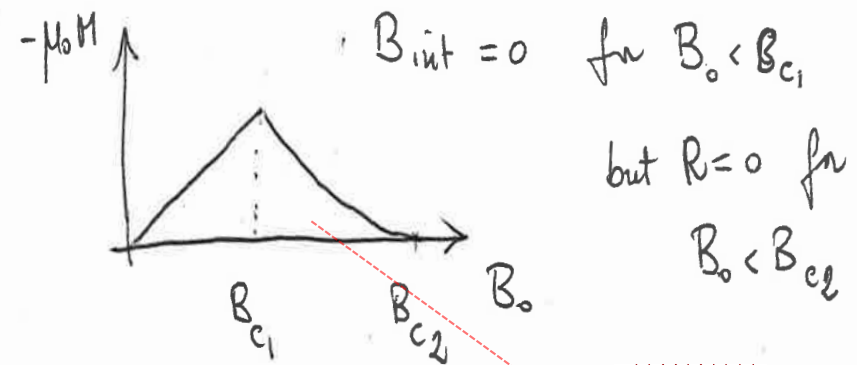
Magnetic flux is excluded from the bulk of superconductors by screening currents flowing at the surface, within the London penetration depth,  $\lambda$

The behavior of superconductors is determined by the ratio between  $\lambda$  and the coherence length  $\xi$ , the distance over which superconducting state can change

$$\lambda < \xi / \sqrt{2} \Rightarrow \text{Type I}$$



$$\lambda > \xi / \sqrt{2} \Rightarrow \text{Type II}$$

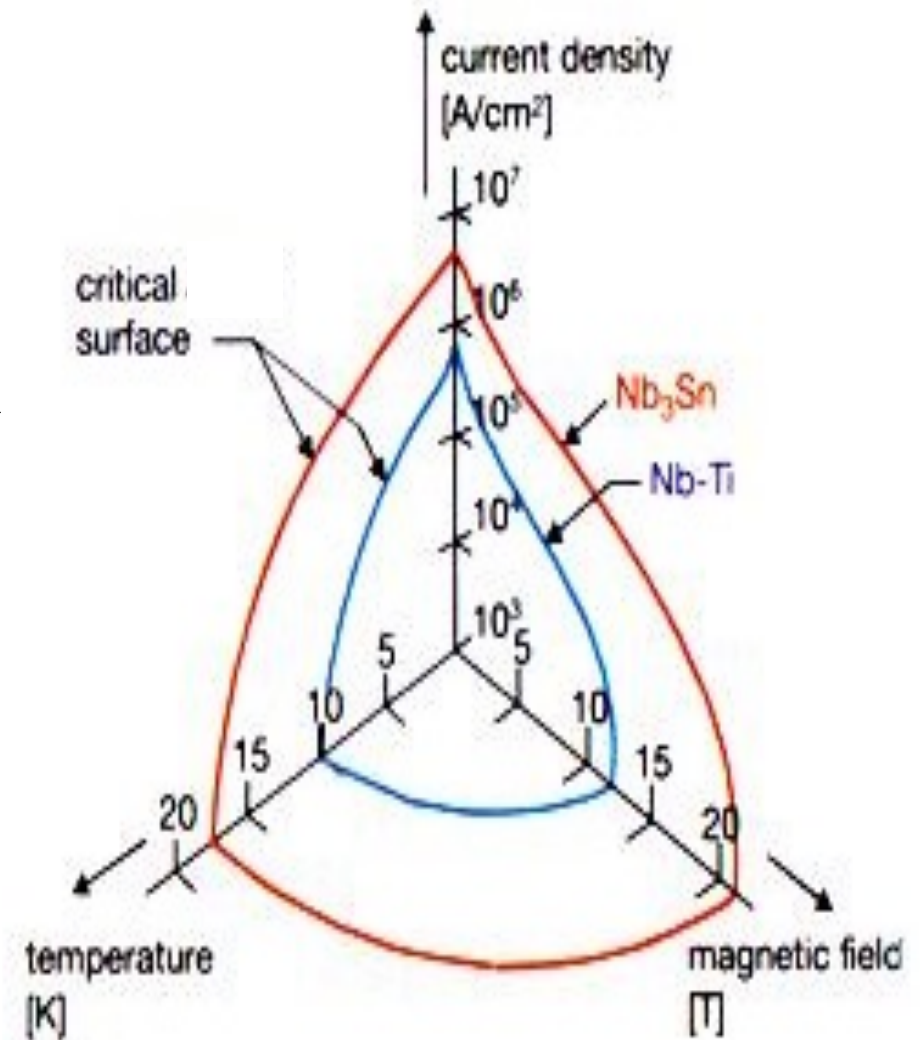


normal islands (size  $\sim \xi$ ) immersed in a sea of SC ( $R=0$ )

Low  $B_c$  values for Type I SCs prevent their utilisation for fusion magnets

Fusion magnets are based only on Type II SCs and are in mixed magnetic state

For  $R$  to drop to zero for temperatures below  $T_c$  and magnetic fields below  $B_{c2}$ , the current density must also be below a critical value,  $J_c$



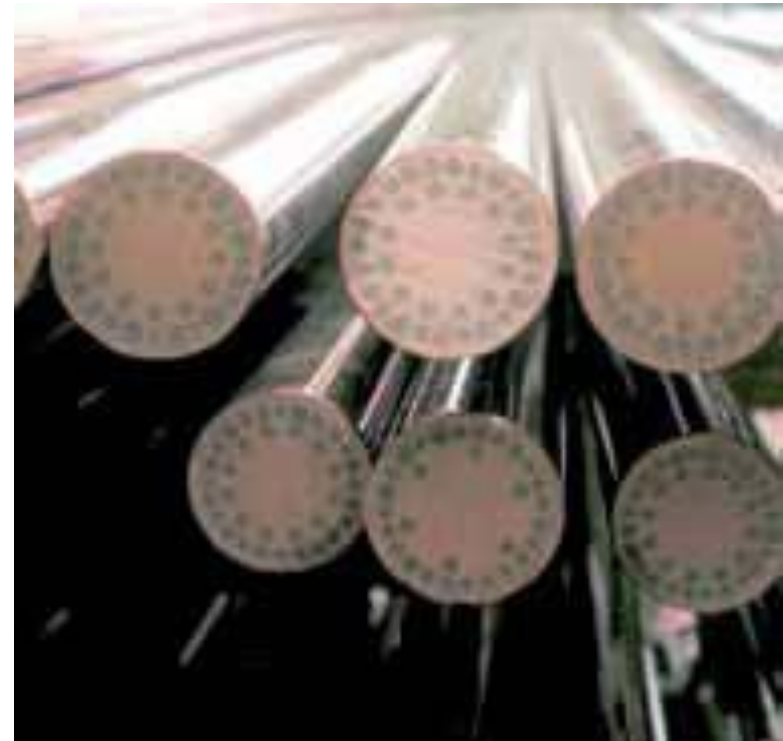
→ critical  $J$ ,  $B$  and  $T$  surface

## NbTi

Typically, the alloy is based on 44% Ti to maximize  $B_{c2}$

$T_c = 9.2\text{K}$ ; magnets up to 8T

Very ductile, co-drawn with copper, produced in thousands of tons mostly for MRI, ~150-200 €/kg



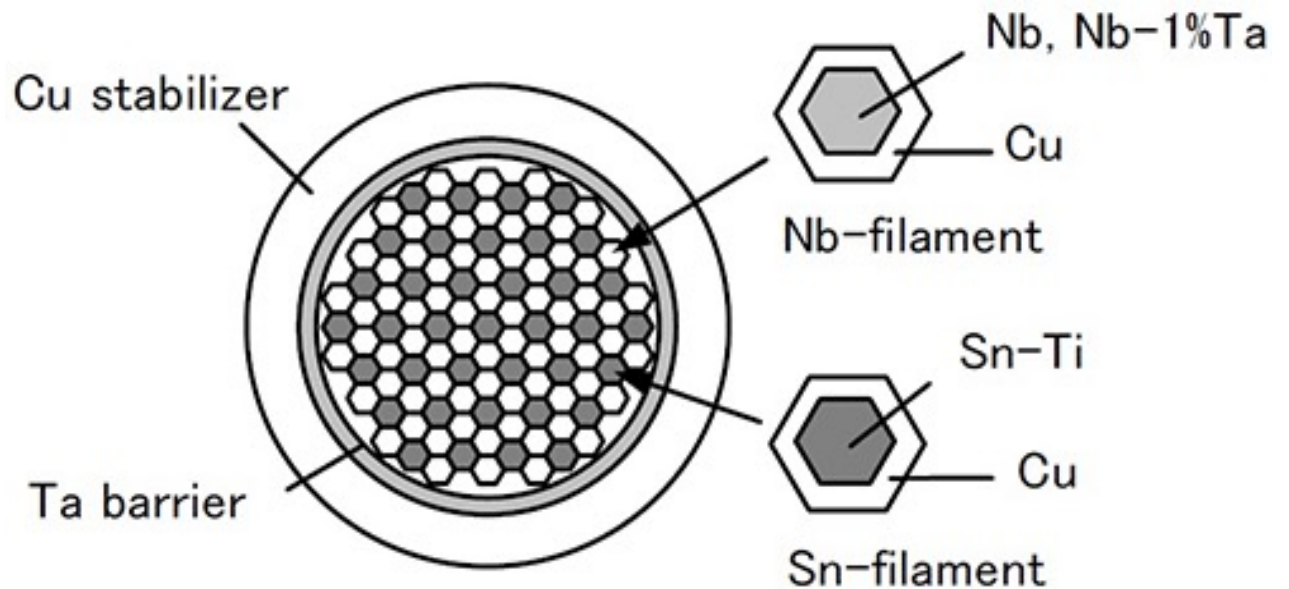
## $\text{Nb}_3\text{Sn}$

Intermetallic compound created by solid state diffusion of Sn into Nb;  $T_c = 18\text{K}$ ; magnets up to 18T

Issues:

$J_c$  strongly decreases under strain (by 30% for 0.5% strain)

Brittle (difficult to wind); limited production, ~600-1000 €/kg



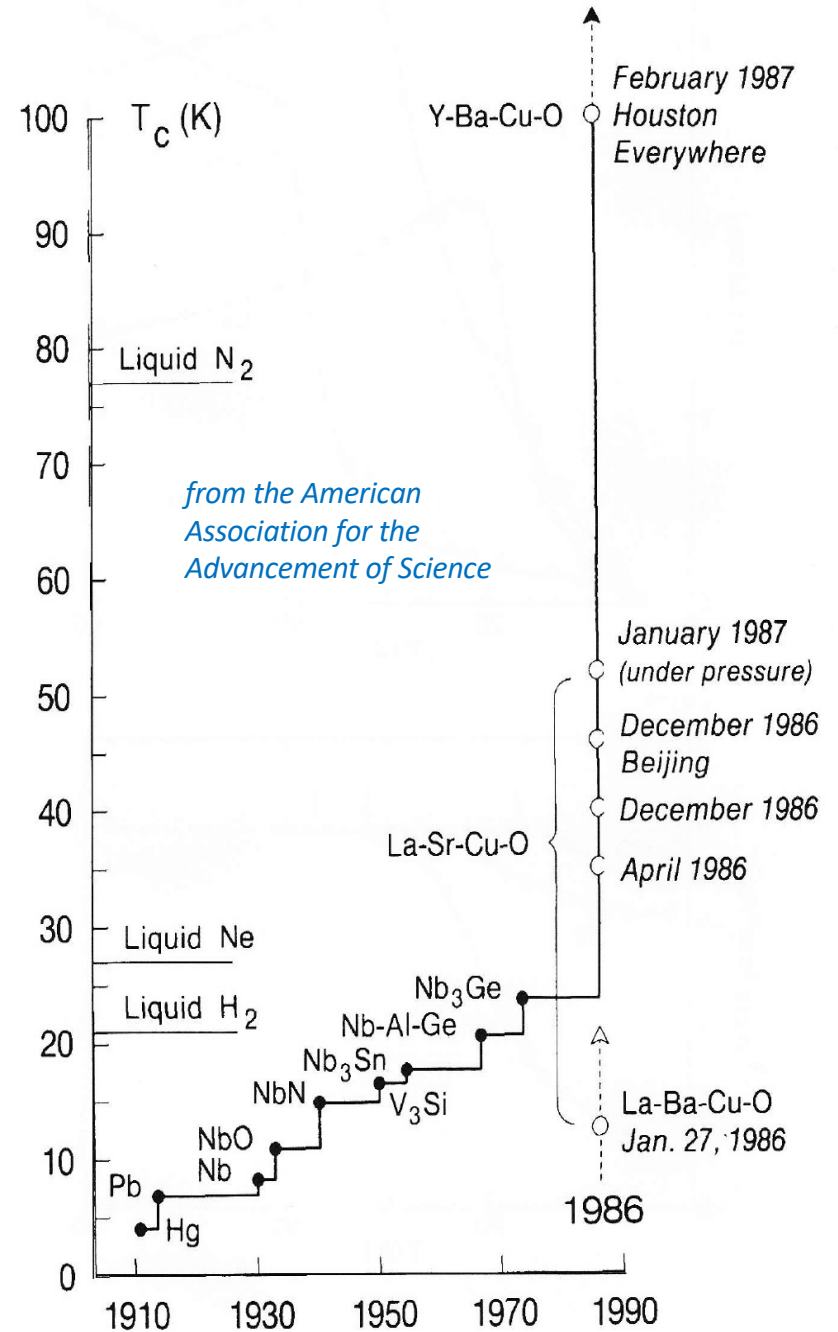
# High temperature superconductivity

In 1986 Bednorz and Müller discovered superconductivity at 30K in  $(\text{LaBa})_2\text{CuO}_4$

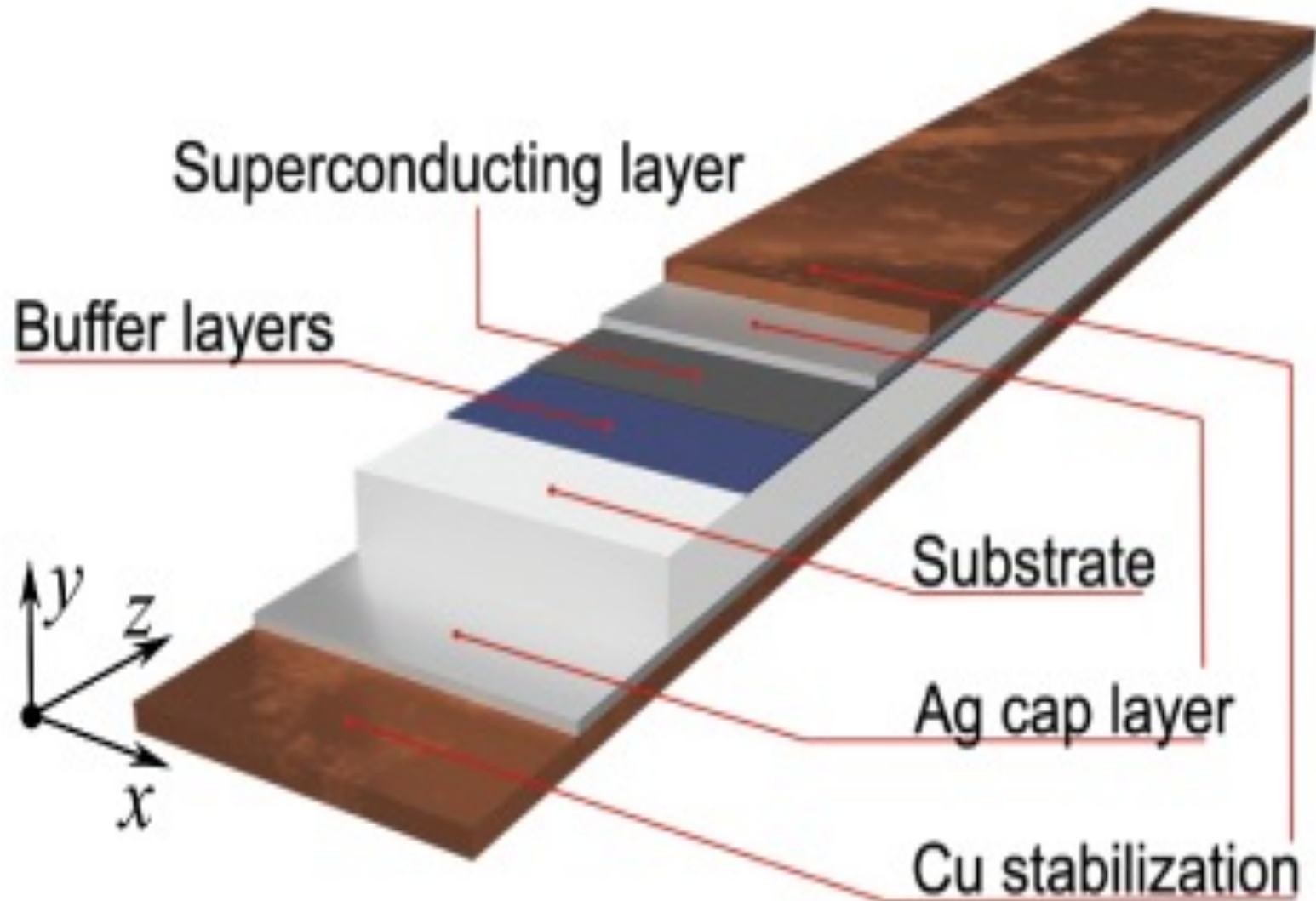
Two classes of HTS materials are potentially suitable for fusion magnets

Bismuth strontium calcium copper oxide compounds ( $\text{Bi2212}$ ,  $\text{Bi2223}$ )

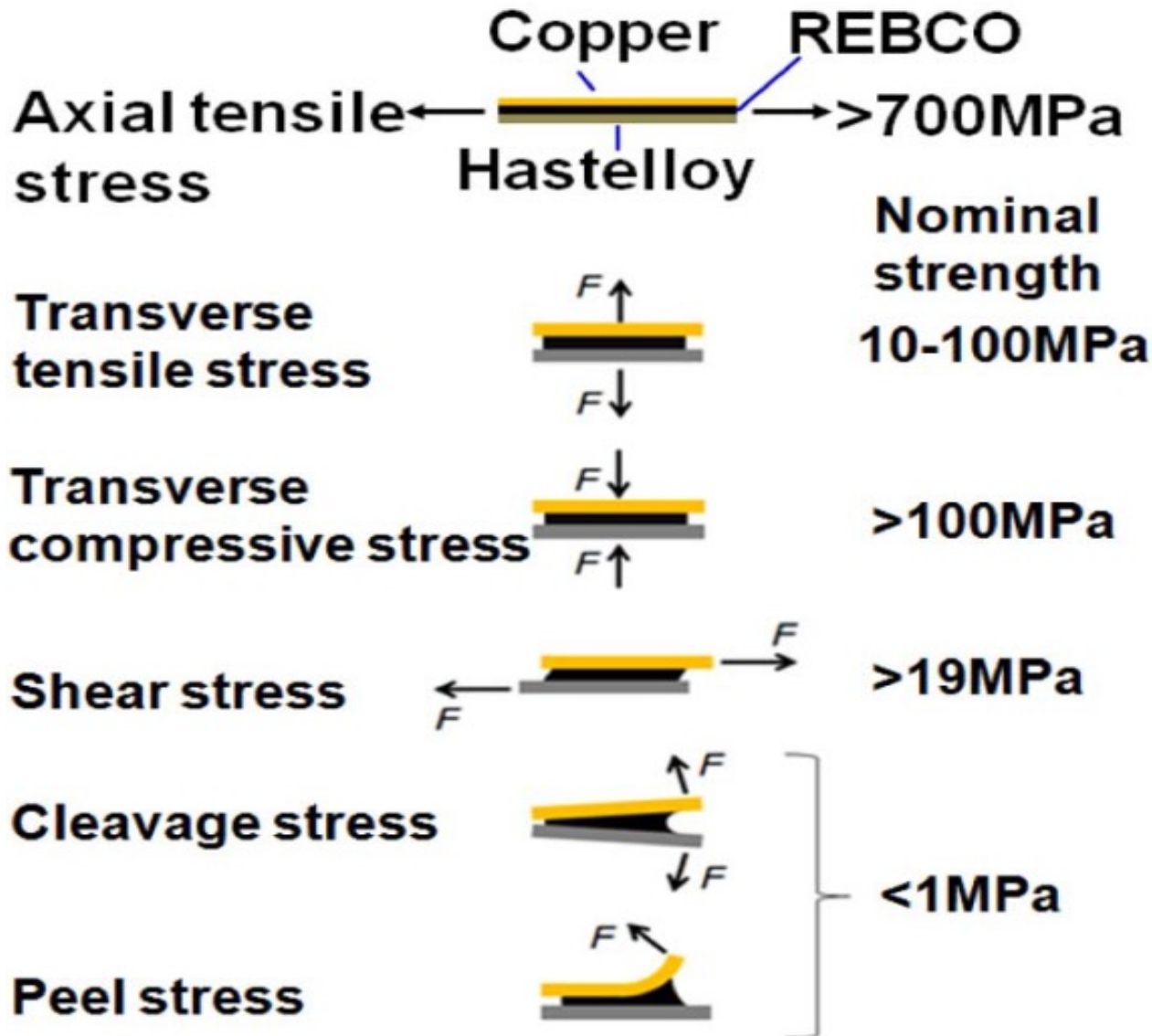
Rare earth barium oxide oxide compounds (ReBCO)



# HTS – REBCO tapes

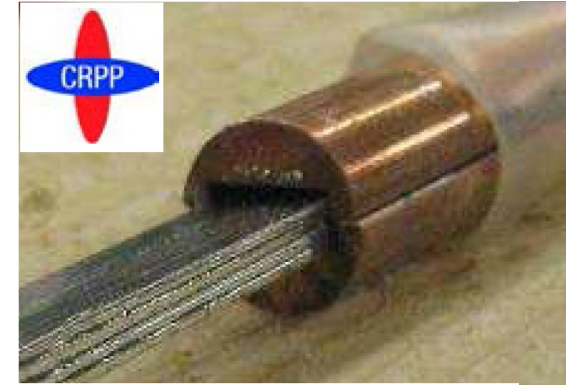
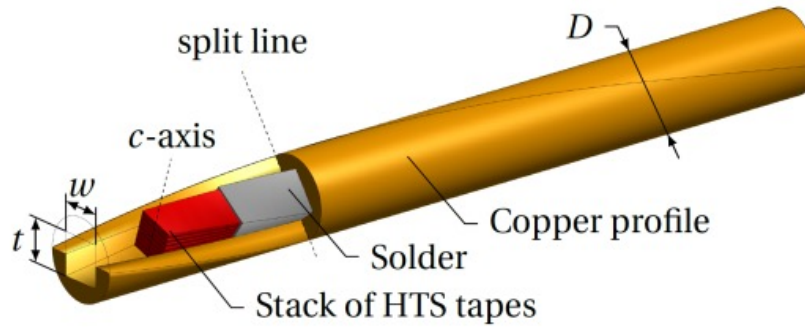
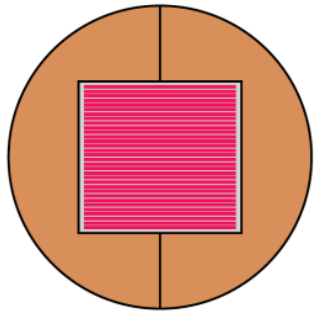


# HTS – REBCO tape mechanical issues

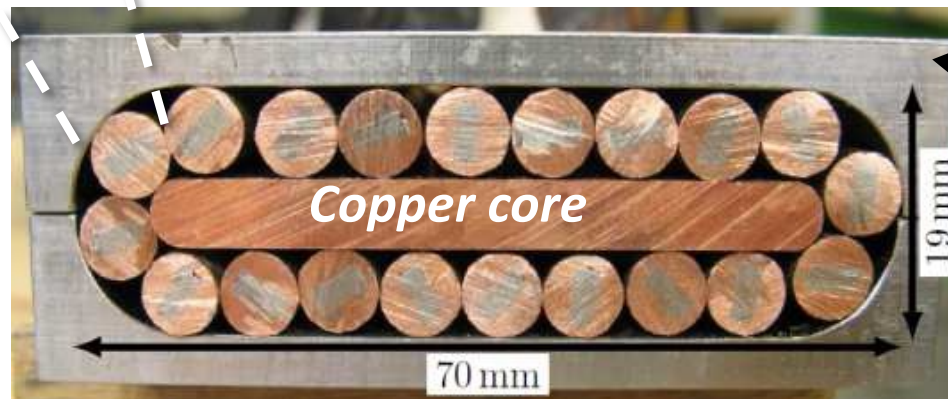
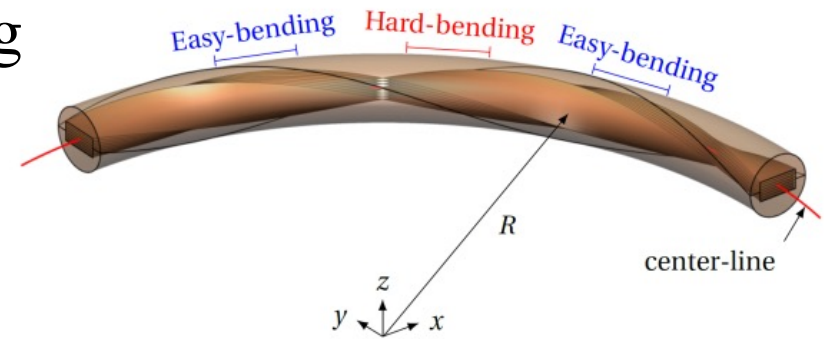


H. Maeda et al., TMS Critical current anisotropy ~ 5 24 (2014) 4602412

# HTS – from tape to cable



twisting and bending



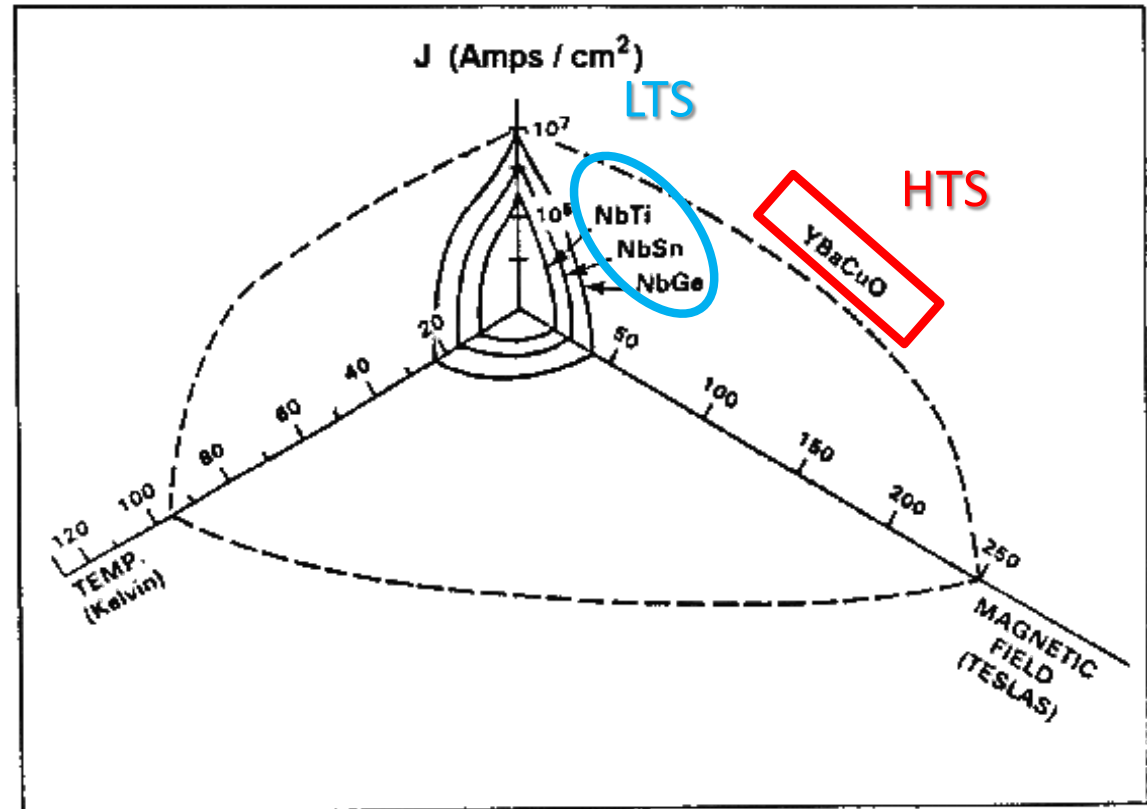
Jacket



# Practical use of HTS

Low B → high temperature

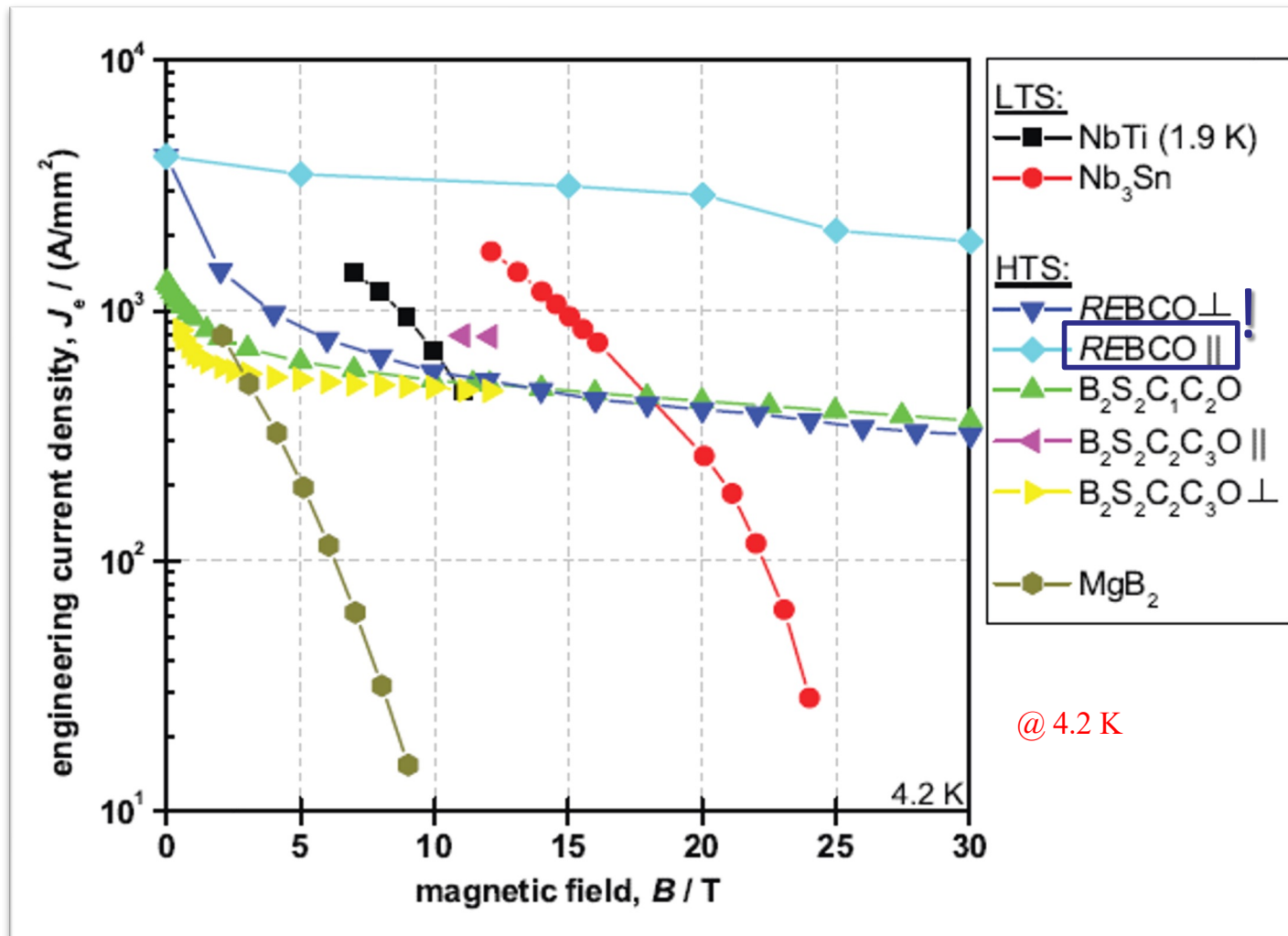
Simpler and cheaper cryogenic systems  
OK for energy transportation



**Phase Diagram**

But for fusion we need high B → low temperature (4.2 K ?)

# Which HTS for fusion?



Courtesy of O.Dicuonzo

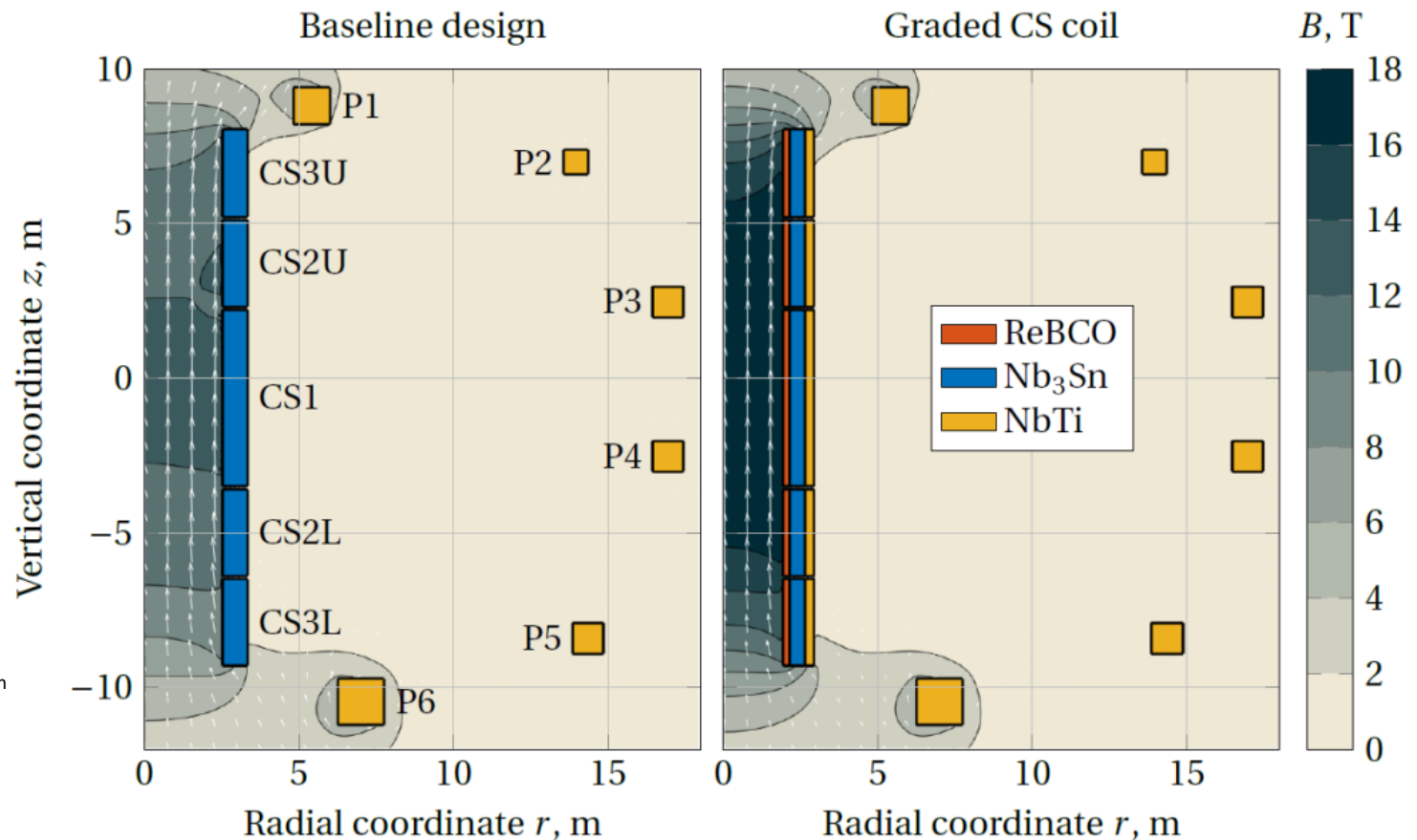
Need high current density at high B → REBCO

# Practical use of HTS - grading

Idea: combine LTS and HTS to take advantage of HTS only where it is needed, i.e. in the higher field part of the magnet

Ex. for DEMO central solenoid

For the same volume, the flux can be increased, or for the same flux the volume (i.e. the cost) can be decreased



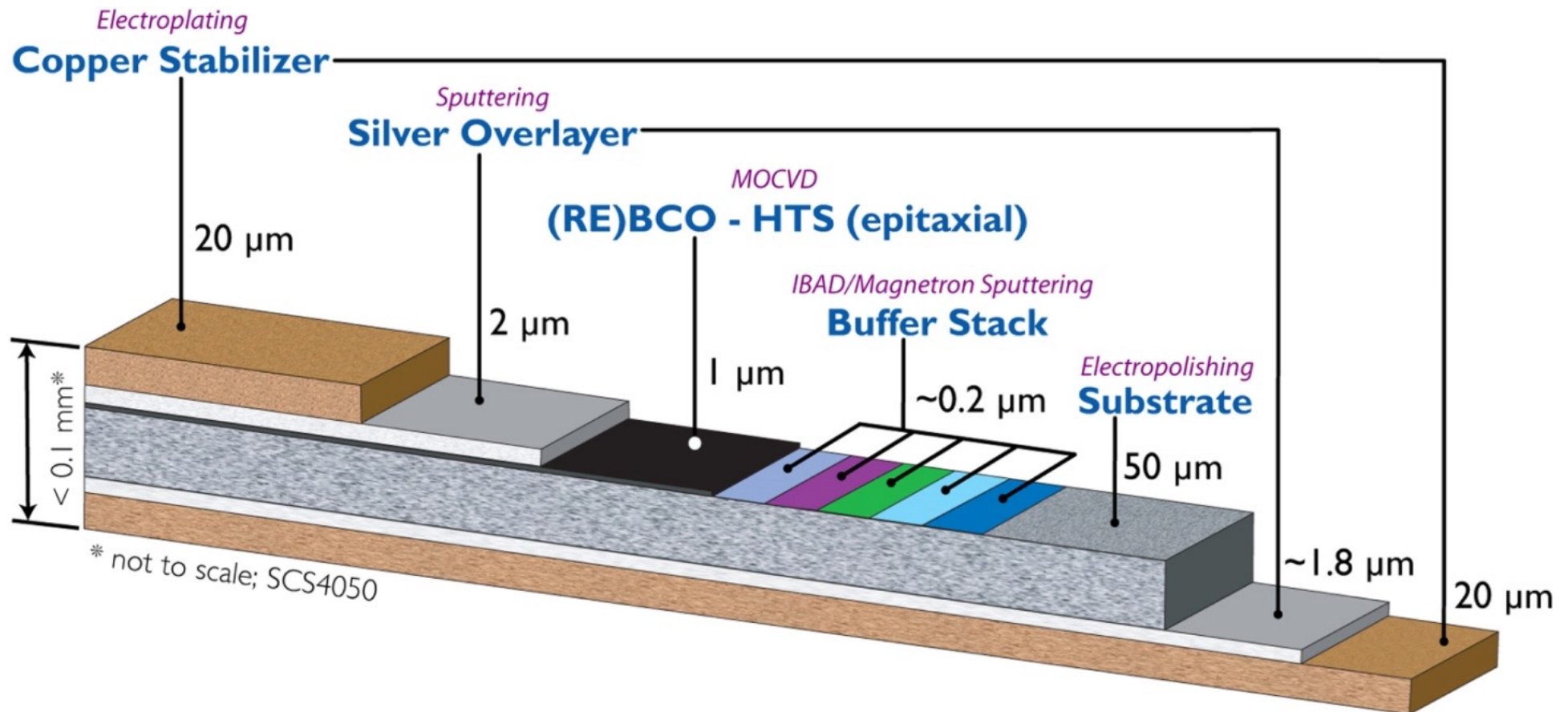
N. Bykovsky, "HTS high current cable for fusion application", PhD thesis

**HTS (YBCO)**

Ceramic thin film on tape

$T_c \sim 100\text{K}$ ; at low temperature withstands fields up to 50T

Limited industrial production,  $\sim 12\text{-}17\text{ k€}/\text{kg}$



Cables based on NbTi or Nb<sub>3</sub>Sn consist of small strands formed by thin SC filaments ( $\varnothing \sim 50\mu\text{m}$ ) inside a Cu matrix

*Why do we need copper ?*

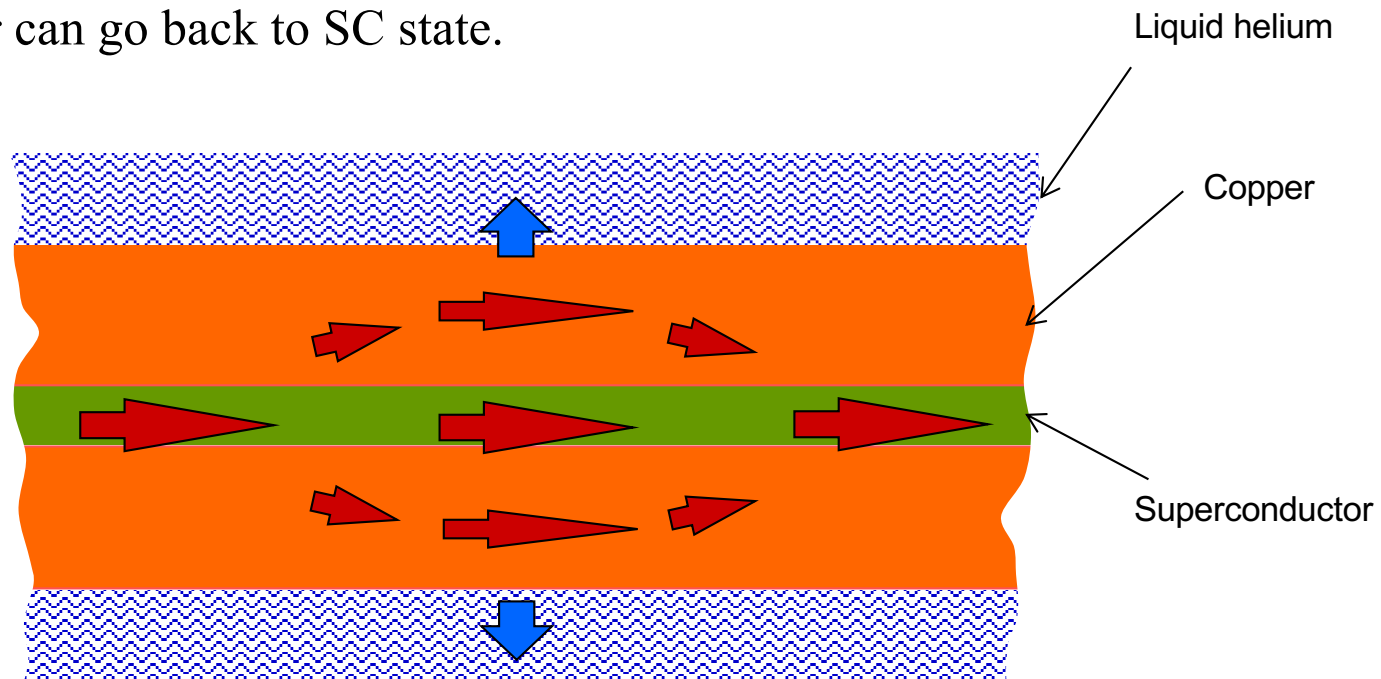


# High current cables for fusion magnets

Cables based on NbTi or Nb<sub>3</sub>Sn consist of small strands formed by thin SC filaments ( $\varnothing \sim 50\mu\text{m}$ ) inside a Cu matrix

## *Why do we need copper ?*

Cu is needed as disturbances may cause a short portion of the SC to become normal. The current then flows in the Cu section, as Cu has lower resistance than non-SC NbTi or Nb<sub>3</sub>Sn. If the heat generated is evacuated efficiently, the conductor can go back to SC state.



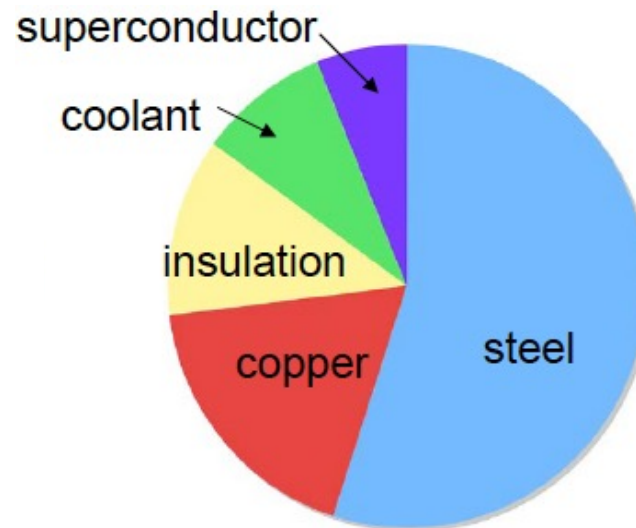
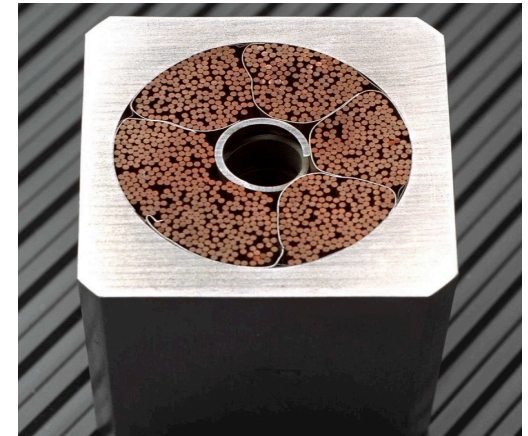
# High current cables for fusion magnets

Cables based on NbTi or Nb<sub>3</sub>Sn consist of small strands formed by thin SC filaments ( $\phi \sim 50\mu\text{m}$ ) inside a Cu matrix



Fusion magnets use large cables with high current because they allow reducing the number of turns, hence the magnet inductance

ITER uses *Cable in Conduit Conductors*. Helium flows between the strands and in the central hole. The steel jacket is needed to cope with the  $J \times B$  force

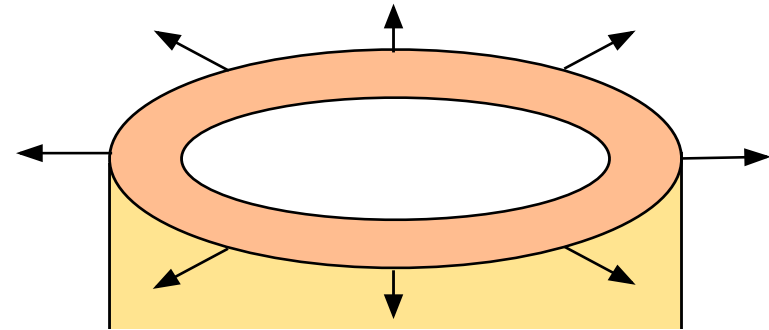


# EPFL Requirements and challenges - Mechanical

Large, high-B fusion magnets experience large electromagnetic loads, thus most of their volume consists of stainless steel

Main loads, all from  $J \times B$  force

Hoop load along the conductor axis,  
 $\sim B \times I \times R$



Solenoid



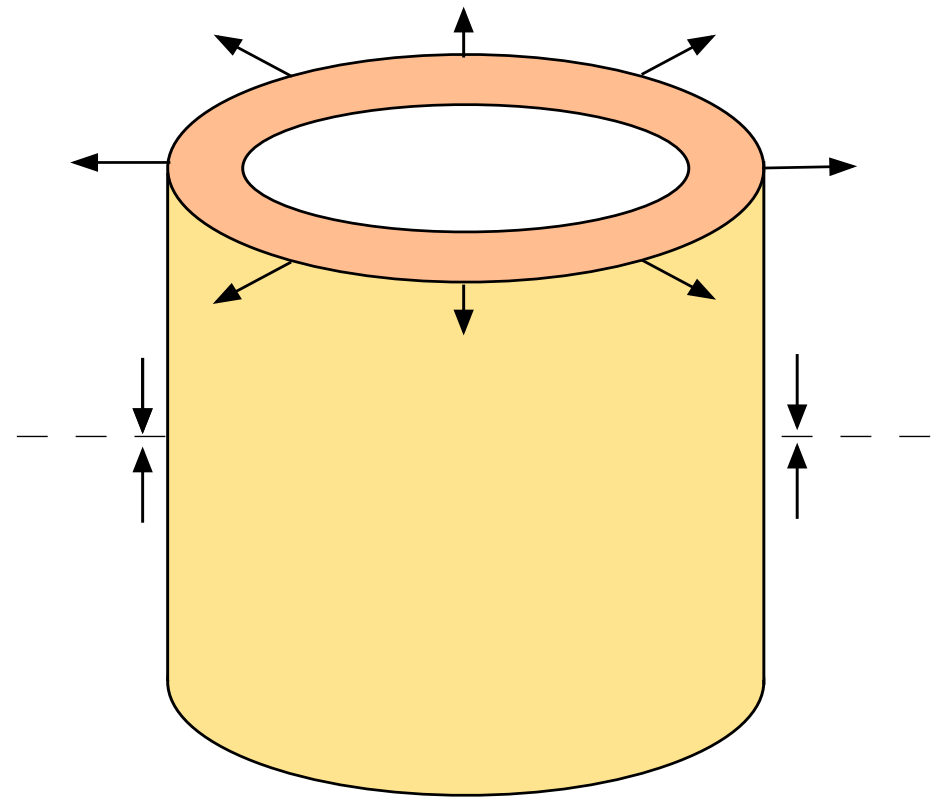
# EPFL Requirements and challenges - Mechanical

Large, high-B fusion magnets experience large electromagnetic loads, thus most of their volume consists of stainless steel

Main loads, all from  $J \times B$  force

Hoop load along the conductor axis,  
 $\sim B \times I \times R$

Vertical load on the coil mid-plane  
(axial compression of solenoid as  $B_r$  is high at the coil ends)



Solenoid

# Requirements and challenges - Mechanical

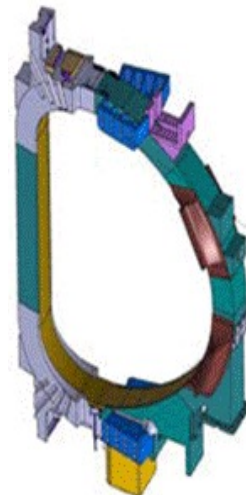
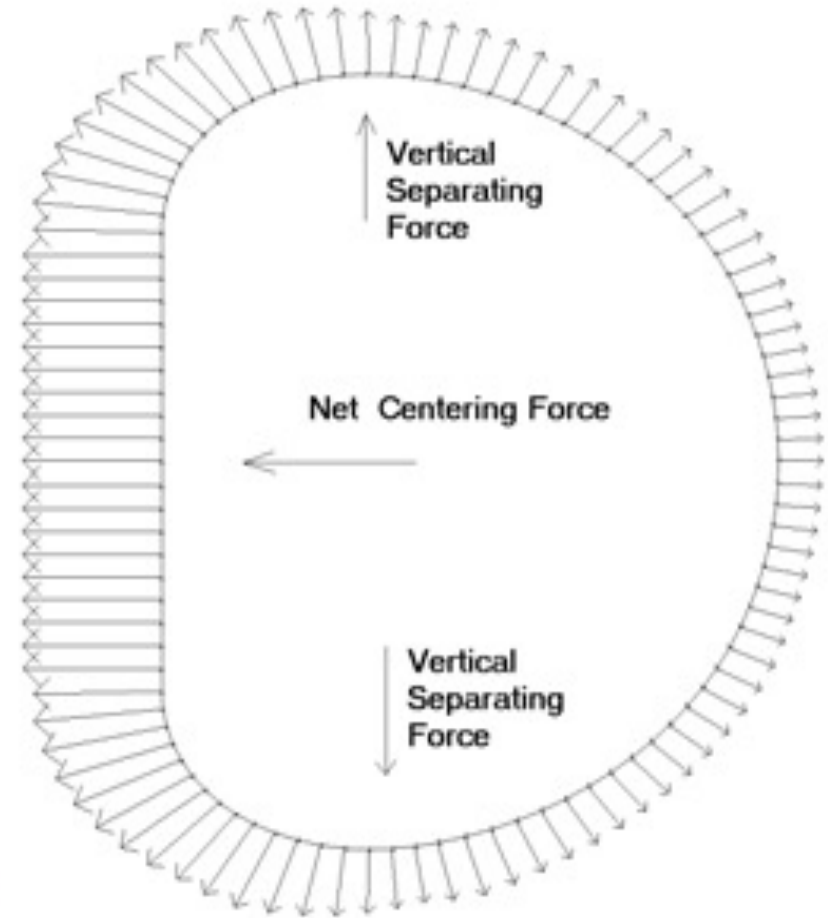
Large, high-B fusion magnets experience large electromagnetic loads, thus most of their volume consists of stainless steel

Main loads, all from  $J \times B$  force

Hoop load along the conductor axis,  
 $\sim B \times I \times R$

Vertical load on the coil mid-plane  
(axial compression of solenoid as  $B_r$  is high at the coil ends)

Centering load on the in-board of non-circular toroidal field coils,  $\sim B \times I$



Non-circular TF coil (e.g. ITER)

# Requirements and challenges - Mechanical

Large, high-B fusion magnets experience large electromagnetic loads, thus most of their volume consists of stainless steel

Main loads, all from  $J \times B$  force

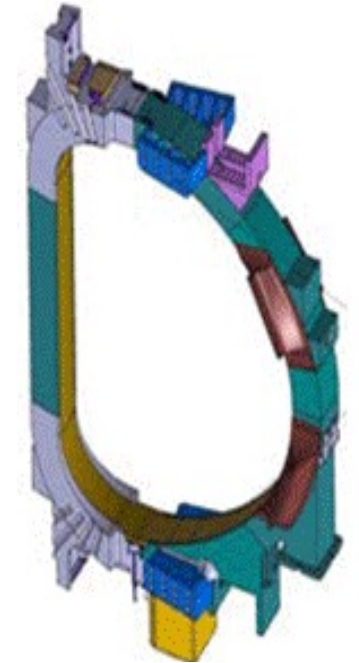
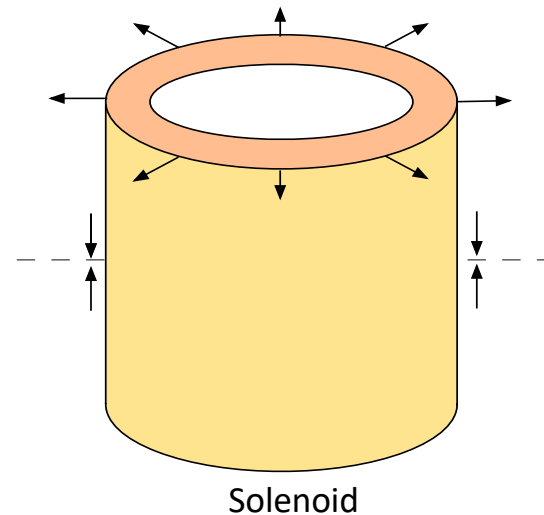
Hoop load along the conductor axis,  
 $\sim B \times I \times R$

Vertical load on the coil mid-plane  
 (axial compression of solenoid as  $B_r$  is high at the coil ends)

Centering load on the in-board of non-circular toroidal field coils,  $\sim B \times I$

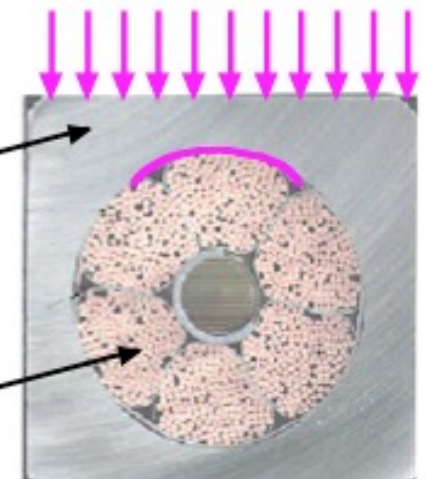
Transverse load accumulation from turn to turn must be avoided for brittle SC ( $Nb_3Sn$  and HTS); for this, a high elastic modulus conduit surrounds the cable

Non-circular TF coil (e.g. ITER)



Jacket material,  
 high modulus  $\approx 200$  GPa  
 big load = high stress,  
 low deflection

$Nb_3Sn$  cable, 33% voids  
 low modulus  $\approx 5$  GPa  
 small deflection = low stress



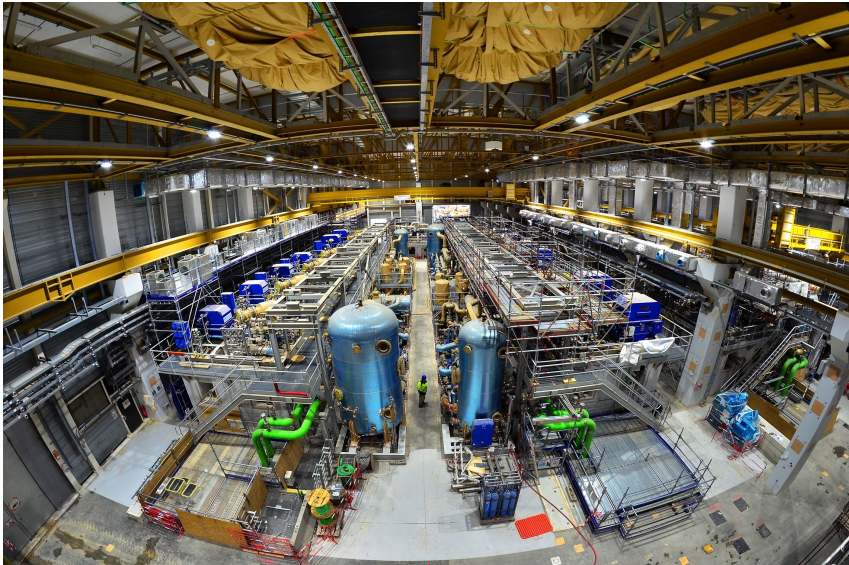
# Requirement and challenges - Thermal

The large mass of the SC magnets is kept cold in a cryostat by large He refrigerators, which use a few tens MW electric power to remove heat load of several tens kW at low temperature

## Main heat loads

- Nuclear radiation on the TF coils
- Ohmic heating of the conductor joints
- Heat conduction (feeders and gravity support)
- AC losses in the coils
- Pumping losses for He circulation
- Heat radiation from room temperature

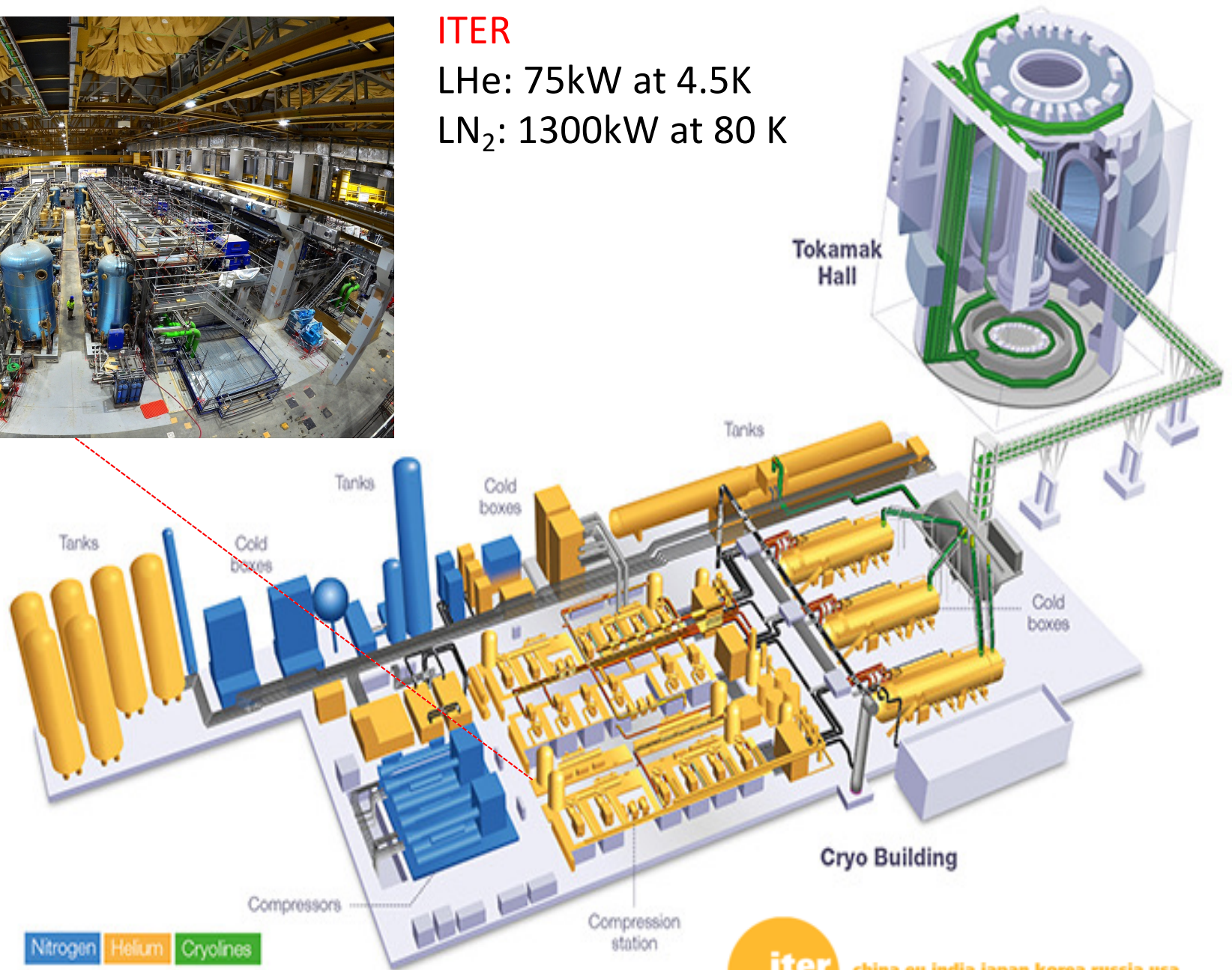
# Requirement and challenges - Thermal



ITER

LHe: 75kW at 4.5K

LN<sub>2</sub>: 1300kW at 80 K



# Requirement and challenges - Thermal

The large mass of the SC magnets is kept cold in a cryostat by large He refrigerators, which use a few tens MW electric power to remove heat load of several tens kW at low temperature

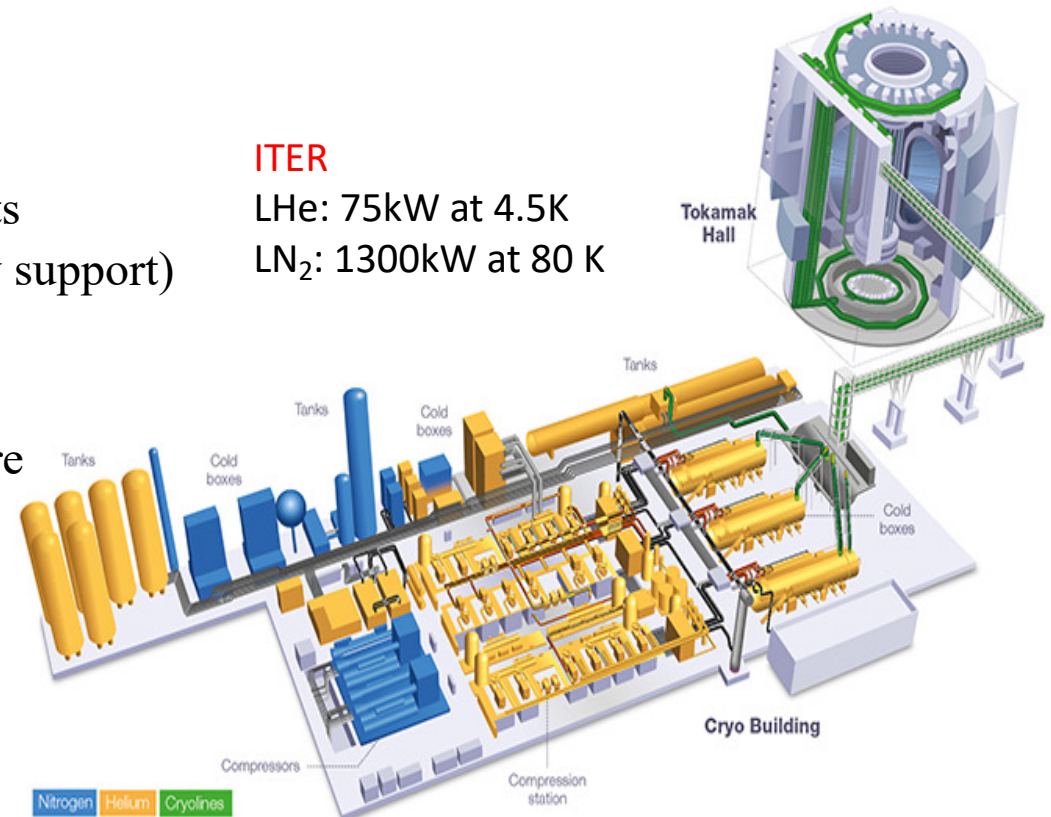
## Main heat loads

- Nuclear radiation on the TF coils
- Ohmic heating of the conductor joints
- Heat conduction (feeders and gravity support)
- AC losses in the coils
- Pumping losses for He circulation
- Heat radiation from room temperature

ITER

LHe: 75kW at 4.5K

LN<sub>2</sub>: 1300kW at 80 K



The variation of the operating temperature must be kept within a temperature margin of  $\sim 1-2$  K

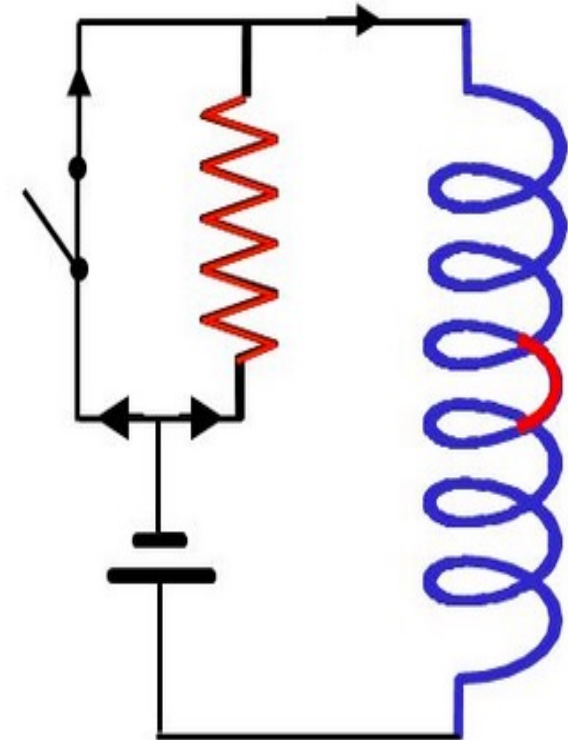
Also HTS also must be cooled below  $\sim 10-20$  K to withstand high fields

# Requirement and challenges - Electrical

In case of *quench* (local, irreversible loss of superconductivity), the SC magnets must be quickly discharged, dumping the large stored energy in external resistors and preventing damage by high temperature spots in the winding

## Main challenges

- 100% reliable, fast quench detection system
- High voltage, high current, fast current breakers
- High voltage insulation for feeders and winding



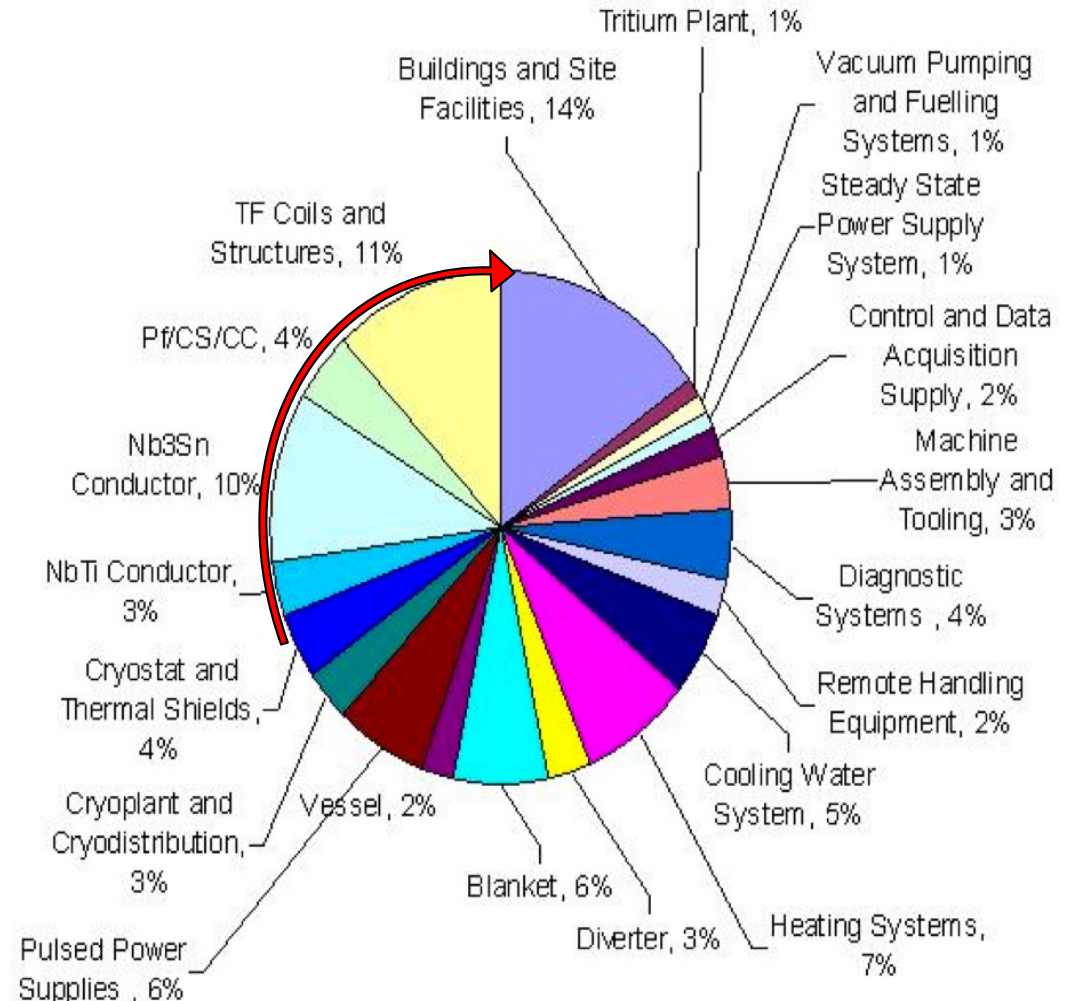
In fusion magnets, electrical insulation consists of Glass-Kapton wraps impregnated by epoxy. The quality of the impregnation is crucial for the coil mechanical integrity and to prevent flashovers to ground during the fast discharge

# Requirement and challenges - Economical

Cost of SC material is  
~100-1000 times that of Cu

SC magnets make up a substantial fraction of the capital cost for a large fusion device, 30% for ITER

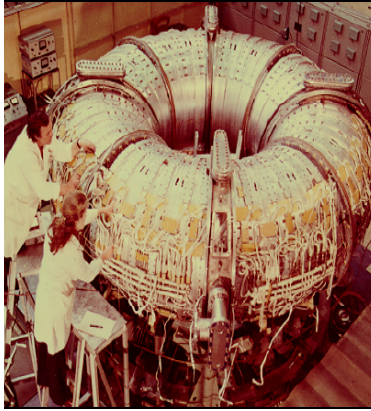
Cost effective design and manufacture of SC magnets are crucial issues on the way to commercially competitive fusion reactors



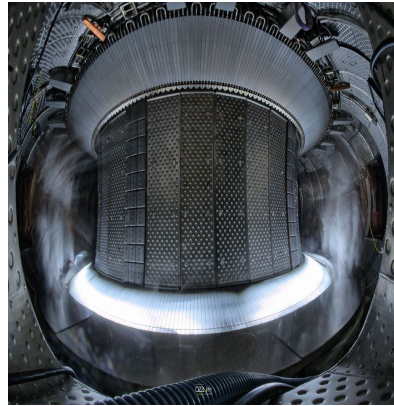


# Present fusion devices with sc coils

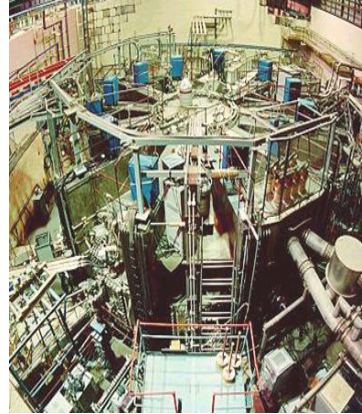
T 7 at Kurchatov -1977  
NbTi, He forced flow, 5T



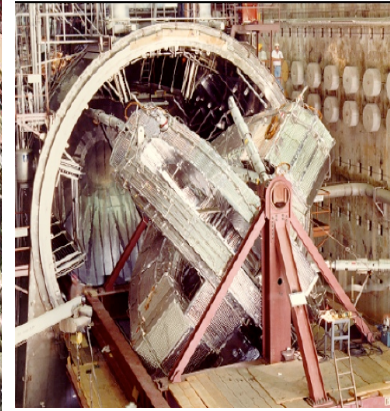
WEST at CEA -2017  
NbTi, He bath, 9T



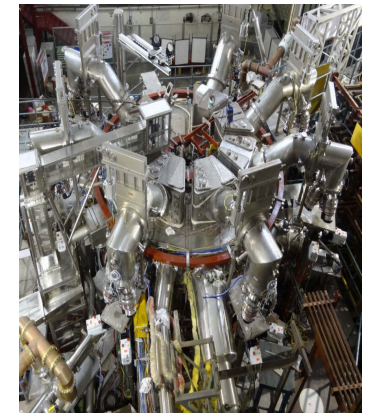
T 15 at Kurchatov -1983  
Nb<sub>3</sub>Sn, He forced flow, 9.3T



MFTF Livermore -1985  
NbTi/Nb<sub>3</sub>Sn, He bath 12.7T



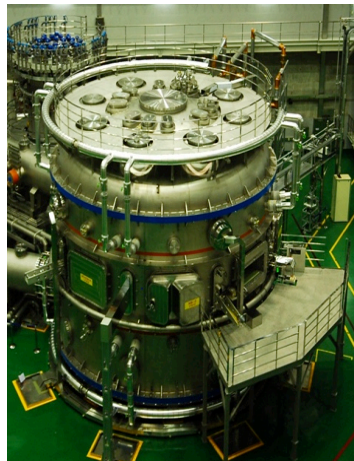
SST1 Bath - 2013  
NbTi, He forced flow, 5T



TRIAM Fukuoka -1986  
Nb<sub>3</sub>Sn, He bath, 11T



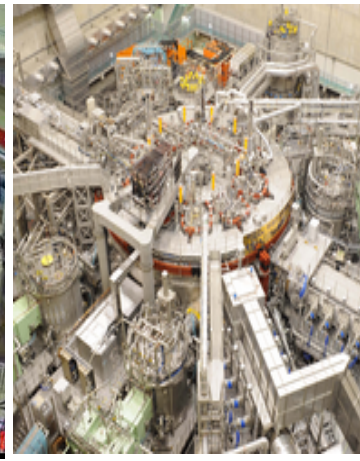
KSTAR- Daejeon 2007  
Nb<sub>3</sub>Sn, He forced flow, 8T



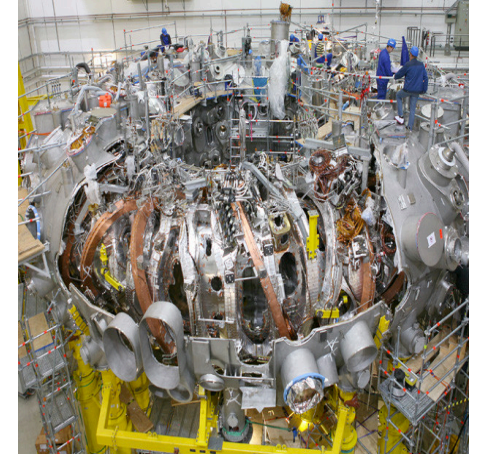
EAST Hefei - 2006  
NbTi, He forced flow, 5.8T



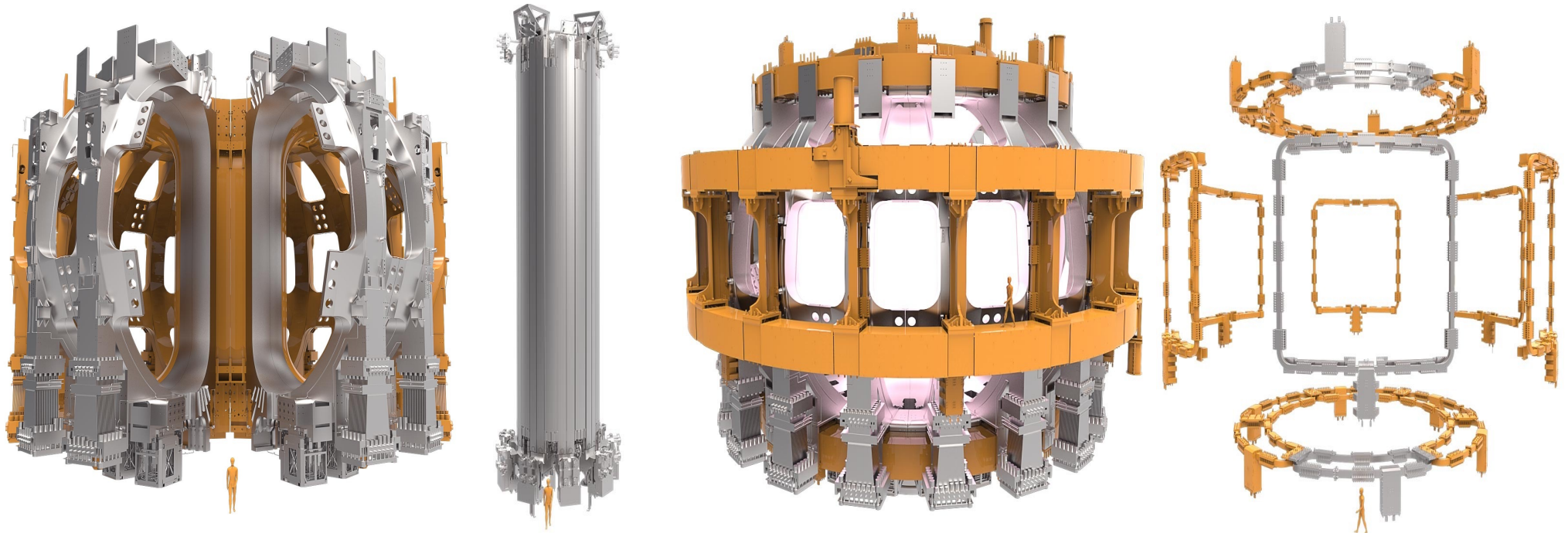
LHD Toki - 1996  
NbTi, He bath, 6.9T



W7-X 7 Greifswald -2016  
NbTi, He forced flow, 6T



# EPFL ITER magnets system – the largest ever built



TF coils  
 $\text{Nb}_3\text{Sn}$ , 11.8T

Central solenoid  
 $\text{Nb}_3\text{Sn}$ , 13T

Poloidal coils  
 $\text{NbTi}$ , 6T

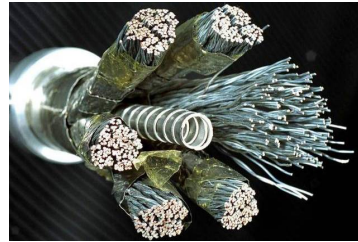
Correction coils  
 $\text{NbTi}$ , 4.2T

48 SC coils, total stored energy = 51GJ

Cooled with supercritical He at 4K

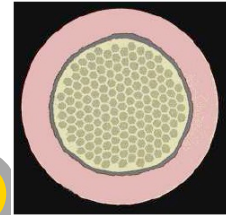
$\text{Nb}_3\text{Sn}$  strand for TF coils and central solenoid: 500 tons, 100'000km

# ITER magnets system – construction



Conductor

## Conductor Manufacture

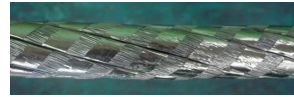


Strand

3rd Stage

1st Stage

Cu Wire



Sub-Wrap

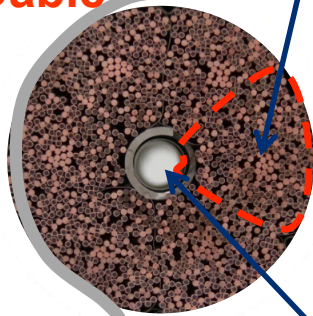
2nd Stage

4th Stage

Cable

Cu Core Cable

Cu Sub-Cable



Wrap

Jacket Assy



Jacket

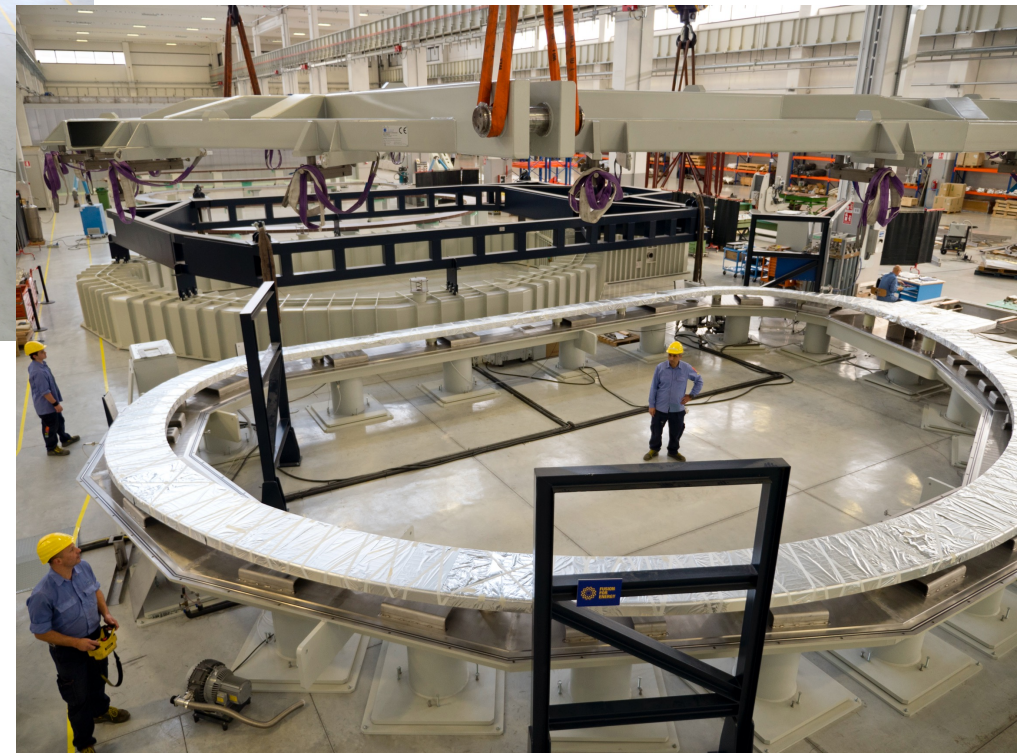


Central Spiral

# ITER magnets system – TF coils



Toroidal Field coils  
winding pack in ASG – La Spezia



# ITER magnets system – TF coils

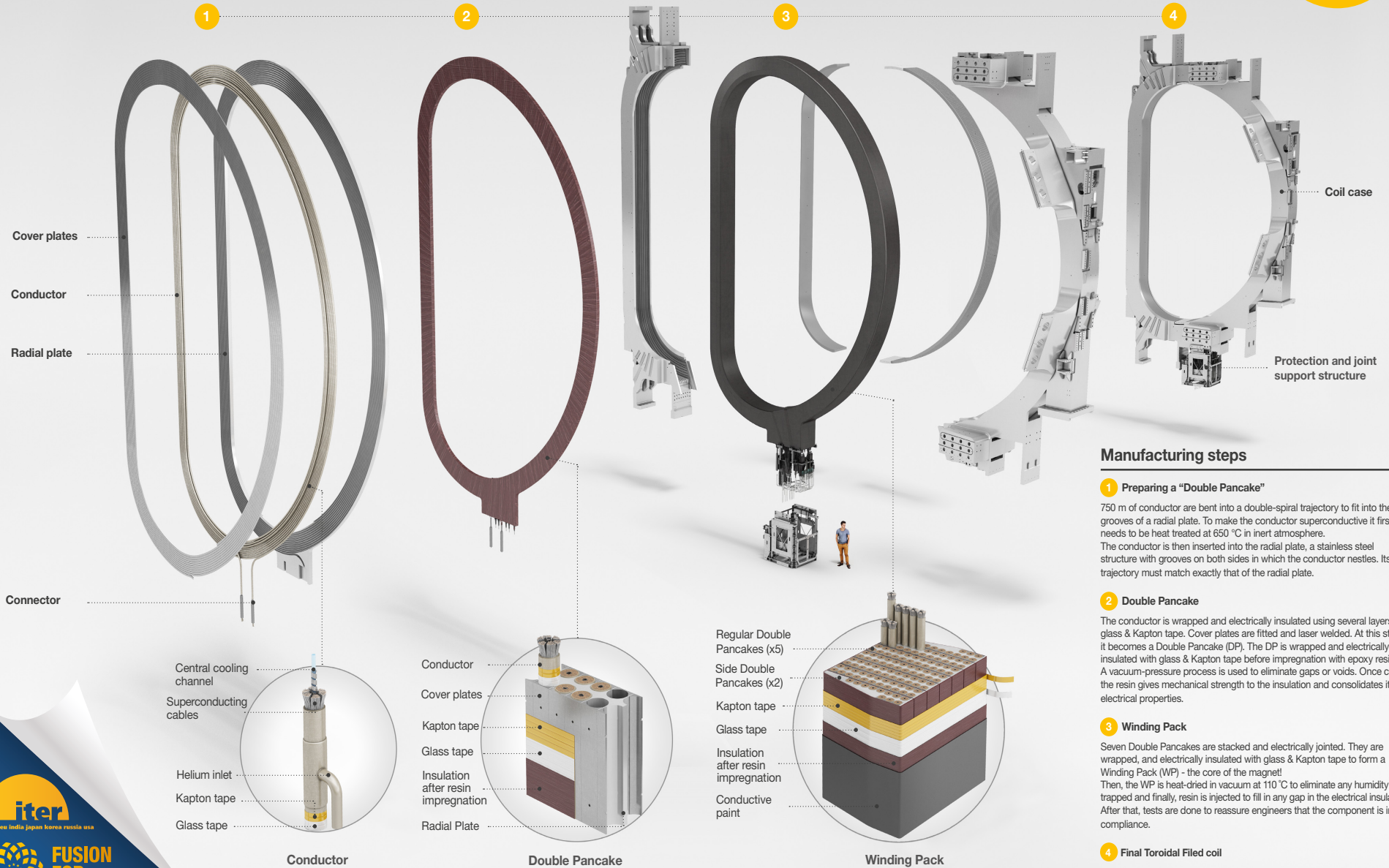


Transporting one  
Toroidal Field coil

# ITER Toroidal Field Coils

18 powerful superconducting magnets will confine the ITER plasma reaching 150 million °C. Powered with 68 000 A they will generate a strong magnetic field of 11.8 Tesla (approximately 1 million times stronger the magnetic fields of the Earth). Europe will manufacture 10 of the TF coils and Japan 8 plus one spare. They will be the biggest Niobium-tin (Nb<sub>3</sub>Sn) magnets ever produced. More than 600 people from 26 companies have collaborated to produce the European TF coils.

Each coil is approximately:  
**14 m high**  
**9 m wide**  
**300 t with its case** - the weight of a Boeing 747



## Manufacturing steps

### 1 Preparing a "Double Pancake"

750 m of conductor are bent into a double-spiral trajectory to fit into the grooves of a radial plate. To make the conductor superconductive it first needs to be heat treated at 650 °C in inert atmosphere. The conductor is then inserted into the radial plate, a stainless steel structure with grooves on both sides in which the conductor nestles. Its trajectory must match exactly that of the radial plate.

### 2 Double Pancake

The conductor is wrapped and electrically insulated using several layers of glass & Kapton tape. Cover plates are fitted and laser welded. At this stage it becomes a Double Pancake (DP). The DP is wrapped and electrically insulated with glass & Kapton tape before impregnation with epoxy resin. A vacuum-pressure process is used to eliminate gaps or voids. Once cured the resin gives mechanical strength to the insulation and consolidates its electrical properties.

### 3 Winding Pack

Seven Double Pancakes are stacked and electrically jointed. They are wrapped, and electrically insulated with glass & Kapton tape to form a Winding Pack (WP) - the core of the magnet! Then, the WP is heat-dried in vacuum at 110 °C to eliminate any humidity trapped and finally, resin is injected to fill in any gap in the electrical insulation. After that, tests are done to reassure engineers that the component is in compliance.

### 4 Final Toroidal Filed coil

Finally, the WP is inserted into a massive stainless steel case, weighing almost 200 tonnes, strong enough to resist the huge forces generated during operation.



www.f4e.europa.eu



# ITER magnets system – PF coils

Poloidal Field coils after successful cold test on ITER site (5 of 6 completed)

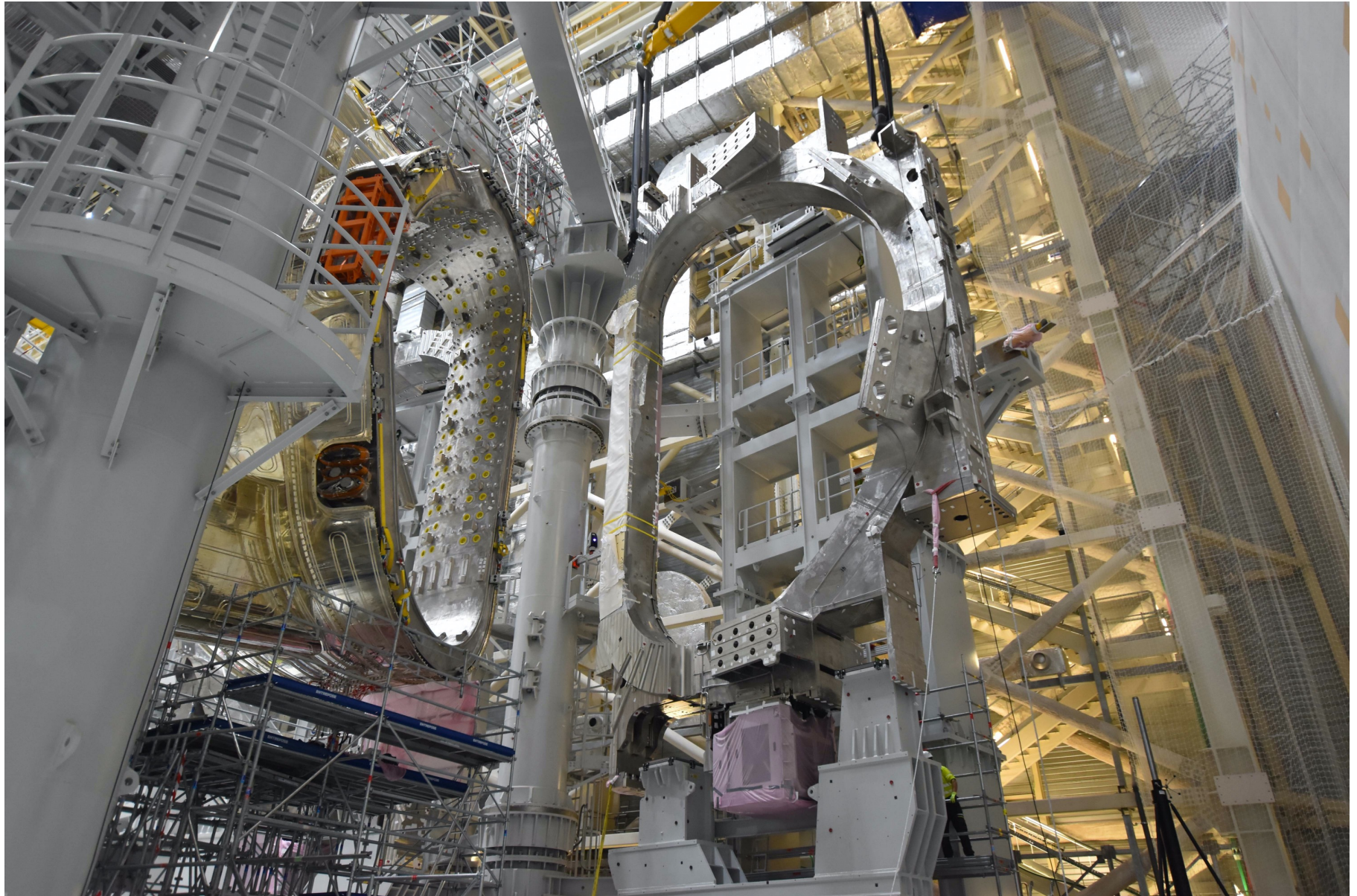


# ITER magnets – installation of 6<sup>th</sup> PF coil





# ITER magnets – installation of 1/18 TF coils with 1/9 of vacuum vessel



# ITER magnets system – the cryostat



# Superconducting magnets for next steps

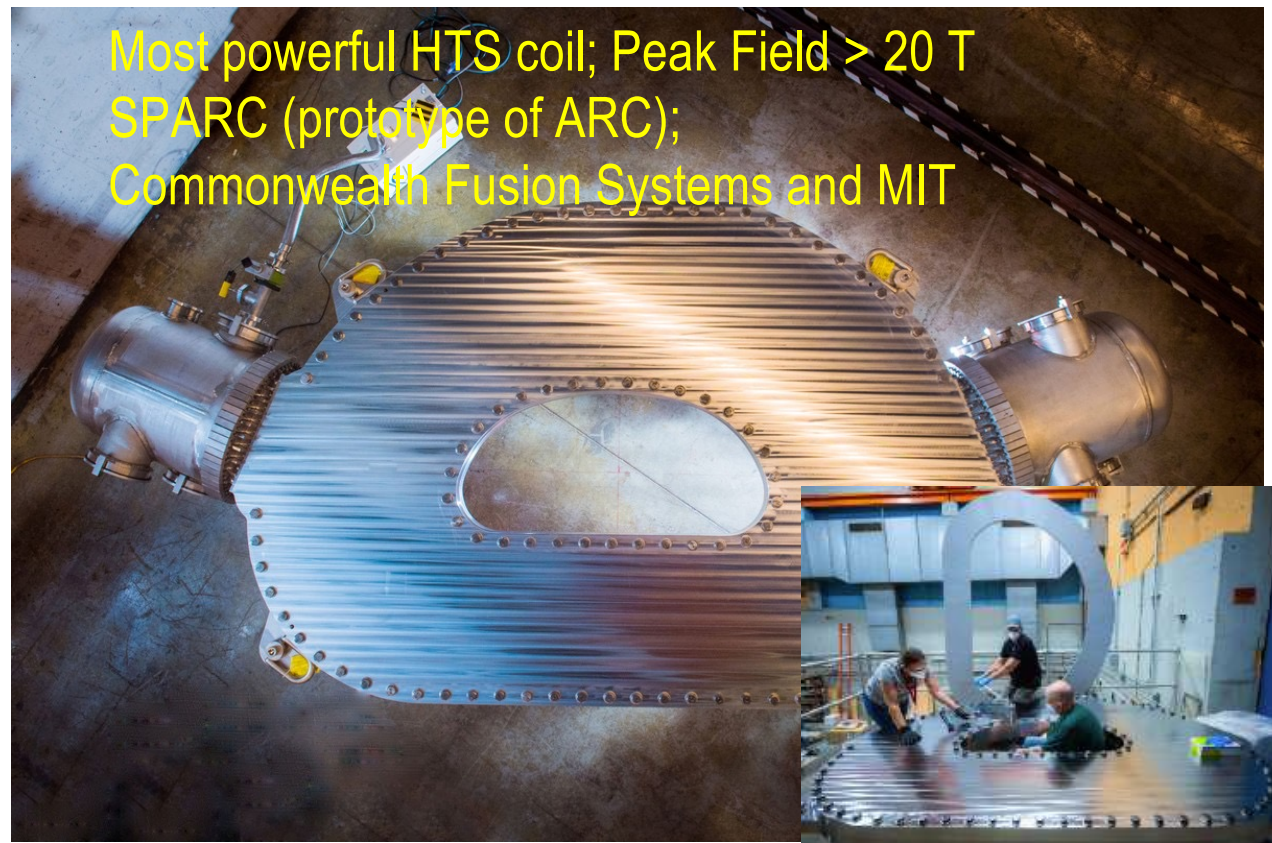
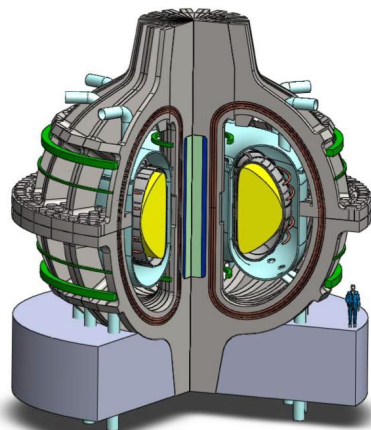
While ITER is being built, several DEMO devices are proposed worldwide, with a broad range of design approaches, from high field tokamak, more compact than ITER, to very large tokamaks with field similar to ITER – all with superconducting magnets

ARC

Major Radius 3.2 m

Peak Field  $\approx$  23 T

HTS coils



Most powerful HTS coil; Peak Field > 20 T  
 SPARC (prototype of ARC);  
 Commonwealth Fusion Systems and MIT

# Superconducting magnets for DEMO

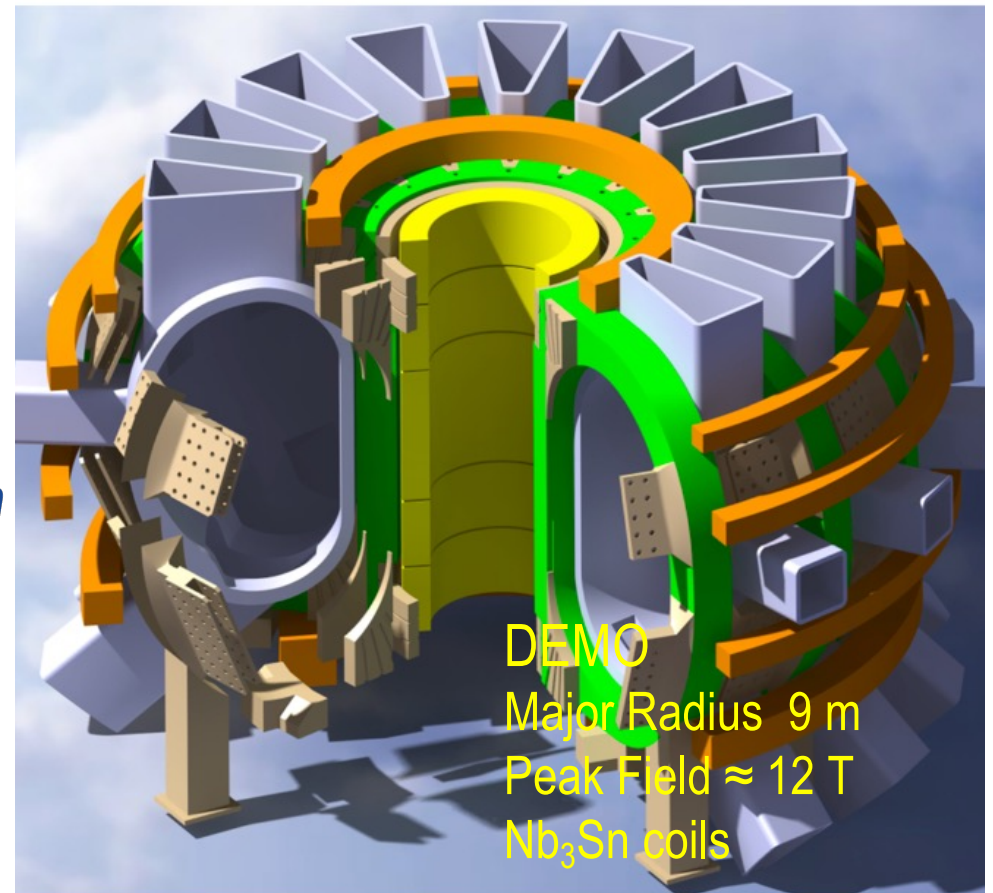
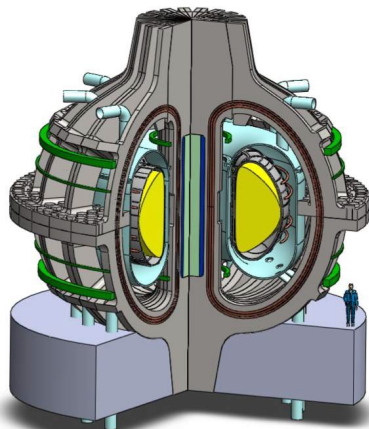
While ITER is being built, several DEMO devices are proposed worldwide, with a broad range of design approaches, from high field tokamak, more compact than ITER, to very large tokamaks with field similar to ITER – all with superconducting magnets

ARC

Major Radius 3.2 m

Peak Field  $\approx 23$  T

HTS coils



# Summary

SC magnets are the enabling technology for steady state fusion reactors

ITER and future fusion devices set new technical challenges for SCs

Cost requirements for power plants constrain design and manufacture

New avenues for compact magnetic fusion reactors can be opened by application of HTS technology

# Selected activities at the SPC superconductivity group

Nikolay  
Bykovskiy

December 11

## Goals of research

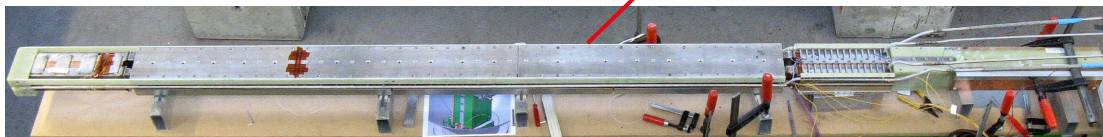
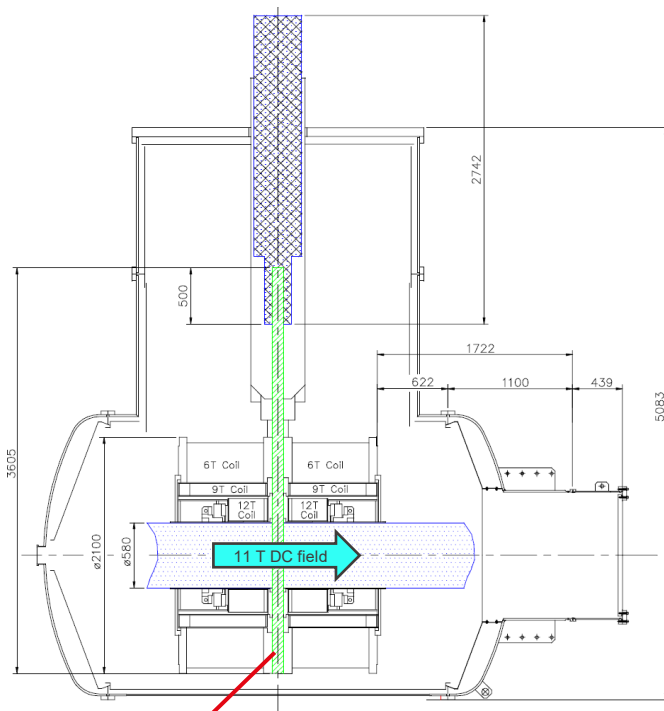
- Contribution to the success of ITER (construction, planning of experiments, ...)
- Improvement of the tokamak concept, understanding of fusion plasmas, and technology for DEMO
- Industrial and societal applications of plasmas

SPC: 9 groups, ~200 people



SPC-SG group, ~20 people





**11 T DC** magnetic field

**~100 kA** operating current in  
**4 – 20 K** temperature range *or*  
**15 kA** current in **4 – 300 K** range

SULTAN sample = pair of ~3.6 m long conductors with joints at the top and the bottom

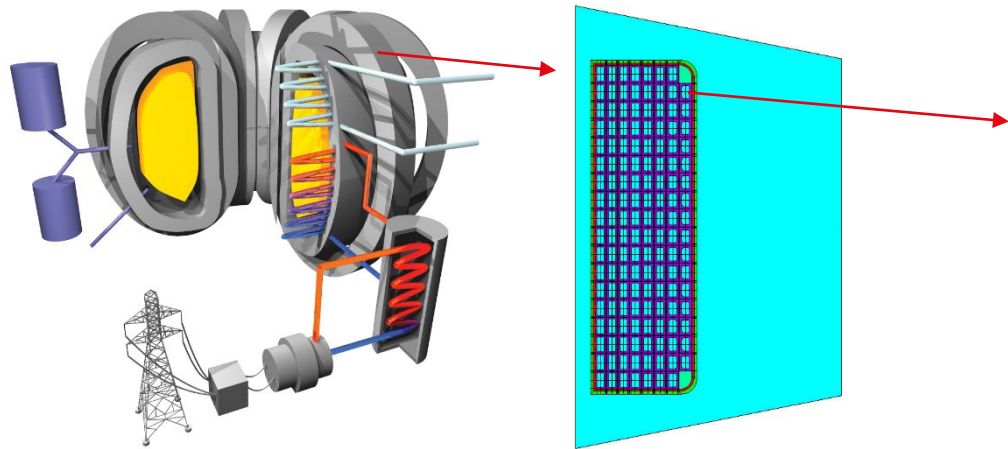
Typical measurements:

- V-T curves at fixed B and I
- V-I curves at fixed B and T
- $0 \rightarrow I_{\text{max}} \rightarrow 0 \rightarrow \dots$  cycling of transport current at fixed B (i.e. EM load cycling)
- AC losses in alternating magnetic field
- Warm-up-cool-down

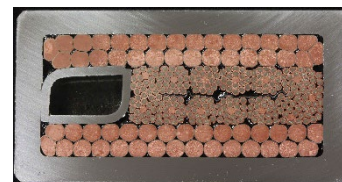
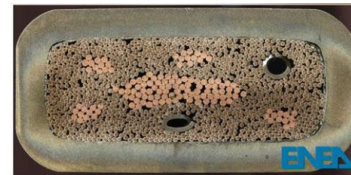
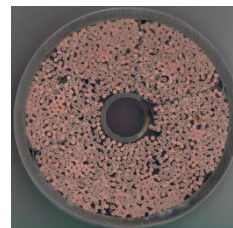
\* other test capabilities: **15 T / 12 T** magnets (80 mm bore); **2 kA / 10 kA PS**; **4 K / 77 K / var T** inserts



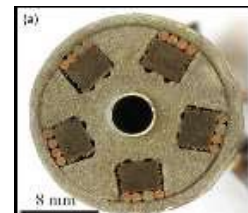
# Superconducting magnets



## LTS wire-based cables



## HTS tape-based cables



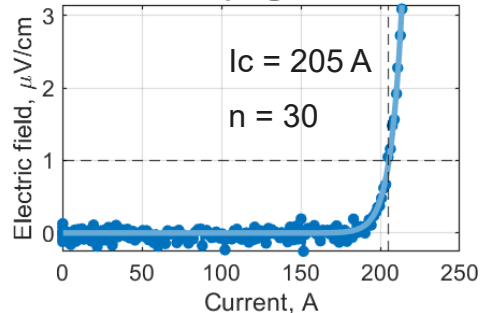
# HTS materials: ReBCO coated conductors



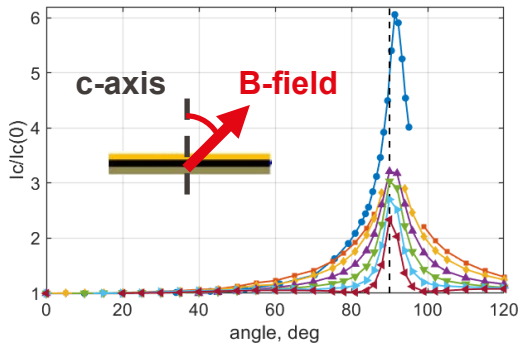
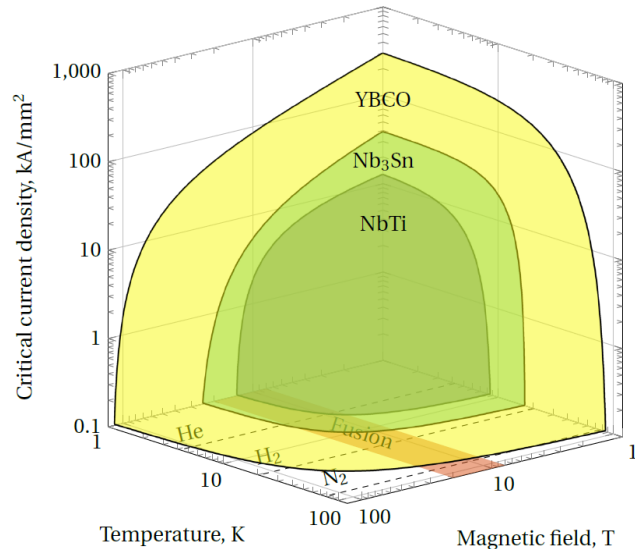
Typical width: 3 mm – 12 mm

Thickness: ~1 μm (sc), ~100 μm (tape)

4 mm wide tape @77 K, self-field

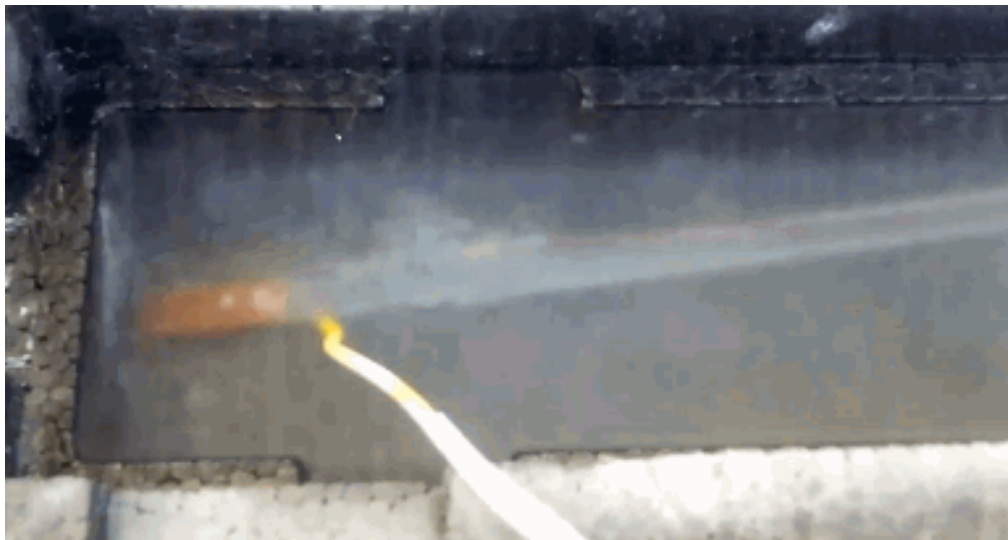


$$E = E_0 \left( \frac{j}{j_c(B, T, \theta, \epsilon)} \right)^{n(j_c)}$$



Thermal runaway ('quench') propagates very slow, thus difficult detection and protection...

- High  $T_c$ , ~100 K
- High  $B_c$ , ~100 T
- High  $j_c$ , ~ $10^{10} \text{ A}/\text{m}^2$
- **Strong anisotropy** of electrical and mechanical properties



Quench propagation velocity (QPV):

~ m/s for LTS

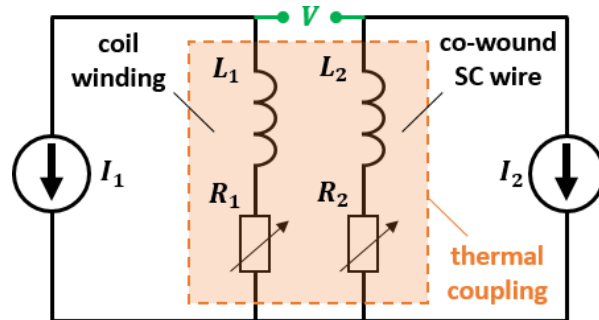
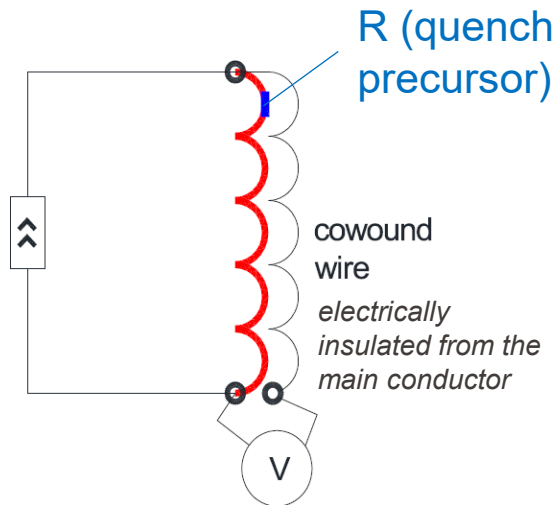
~ cm/s for HTS

$$V_R \sim I_{op} R_1(T_{max}) \cdot QPV \cdot t$$

- $T_{max}$  rises fast,  $V$  responses slow
- Pulsed operation in Tokamaks (inductive voltage ~10 kV!)

Quench detection by measuring coil voltage is problematic for HTS in fusion magnets...

# Quench detection: co-wound wires



aka SQD wires

in good thermal contact with the main conductor:  $T_1(x) \approx T_2(x)$

$$V = V_{R_2} - V_{R_1} + V_{L_2} - V_{L_1} \approx V_{R_2} \gg V_{R_1},$$

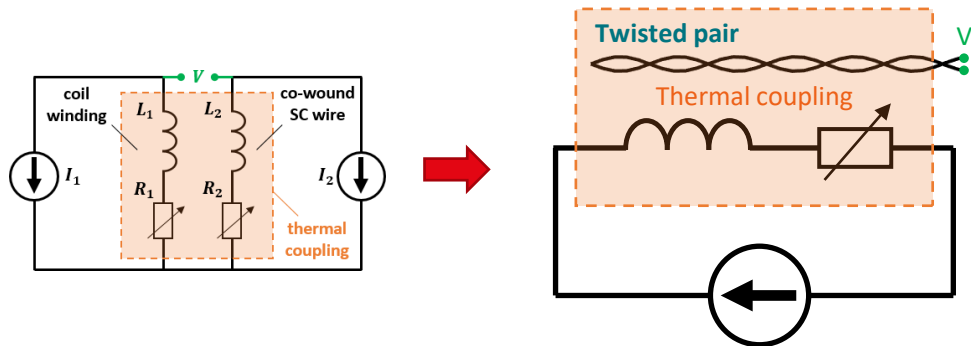
assuming that:

- ❖  $T_c(B_{max})$  of detection wire  $>$  T quench,
- ❖ High voltage along SQD in normal state due to:
  - Material composition (exclude Cu or Ag!)
  - Operating current  $I_2$

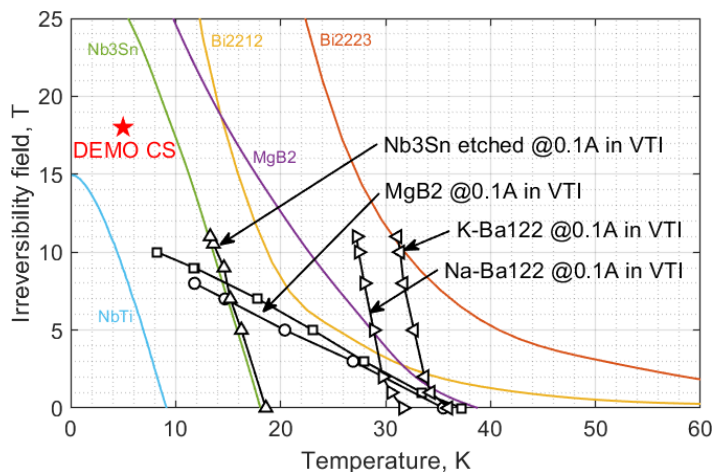
$$V = V_R + V_{L_2} - V_{L_1} \approx V_R$$

How to increase detection sensitivity?

# Quench detection: non-invasive integration



Twisted-pair SQD wires to enhance cancellation of inductive signal + complete electrical insulation



Material selection for SQD wires is broad but limited, i.e. not suitable for certain applications?

→ alternative methods being also developed at SPC, including thermocouple chain and optical fibers...

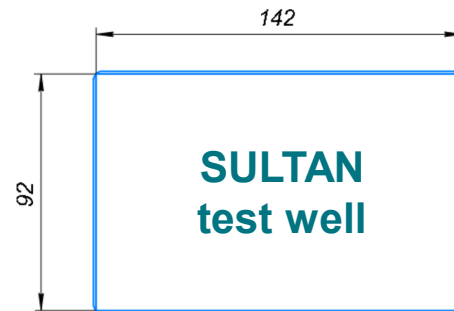
# Quench experiment in SULTAN\* on long lengths

Task objective – development of temperature-based quench detection immune to EM noise and mechanical strain

Pursued options:

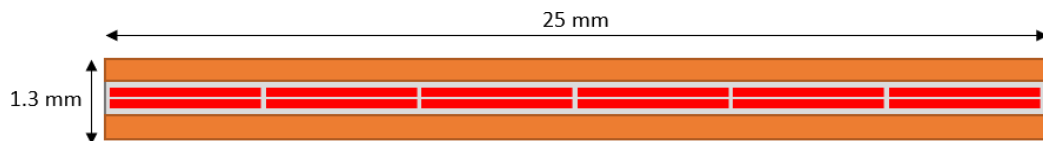
- Superconducting twisted-pair wires (electrical method)
- Shielded thermocouple chain (electrical)
- Mechanically decoupled optical fiber (optical)

→ But first, need for high-current ReBCO insert coil...

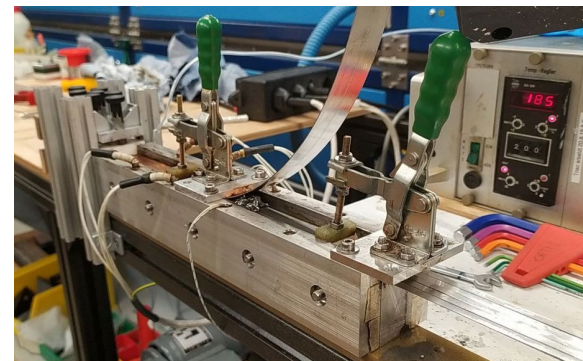


\* direct current drive up to 15 kA / 10 V, thus T range from ~5 K to ~300 K

Laminated stacked-tape soldered conductor (LASSO)



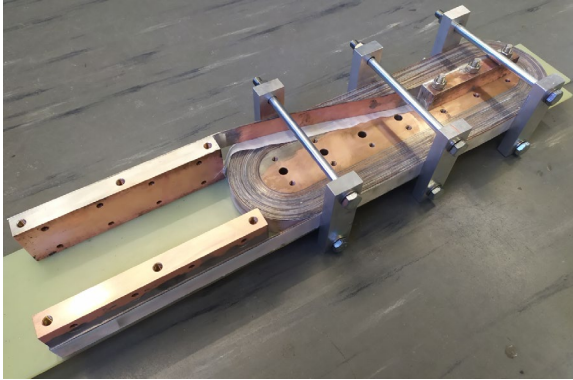
Copper pre-tinning & Cable soldering



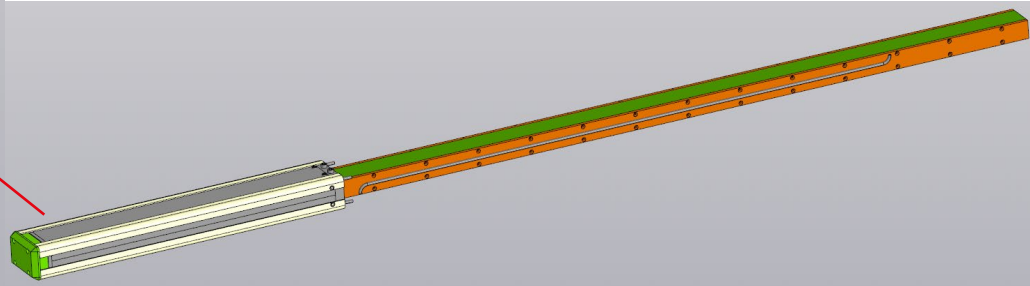
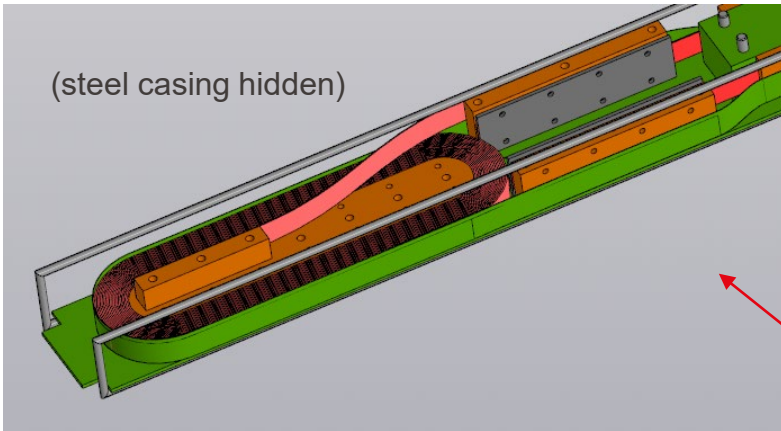
# Cable soldering & coil winding



# SULTAN sample



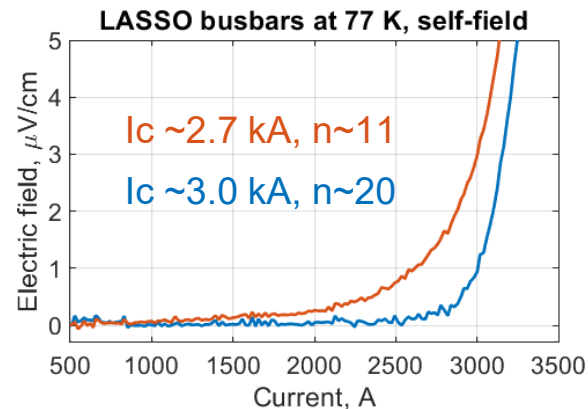
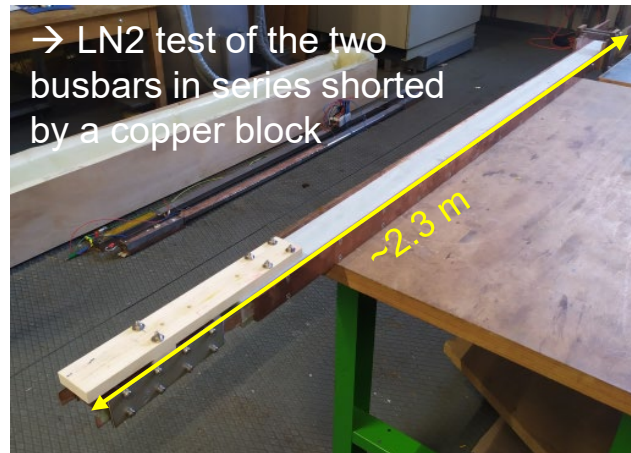
- 20-turn winding,  $R_{min}$  30 mm, no hard bending, fiberglass turn insulation
- Impregnation and indirect cooling by aqueous DMSO solution
- $I_c \sim 12$  kA at 5 K, 11 T (predicted) and  $\sim 1$  kA at 77 K, self-field (predicted & measured)
- Field constant  $\sim 0.2$  T/kA (coil center), Coil inductance  $\sim 0.2$  mH



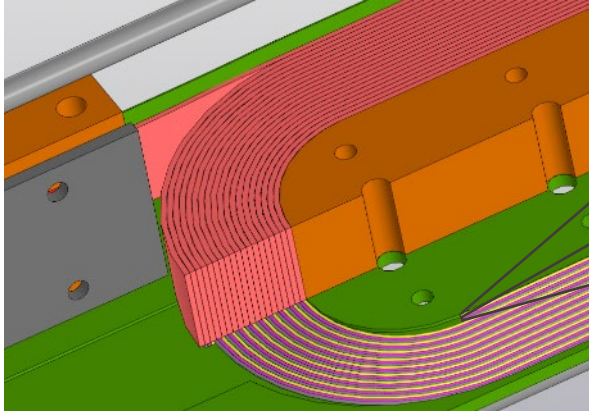


# SULTAN sample: ReBCO busbars

→ Soldering 5-m long LASSO cable made of 25 ReBCO tapes in 4 layers



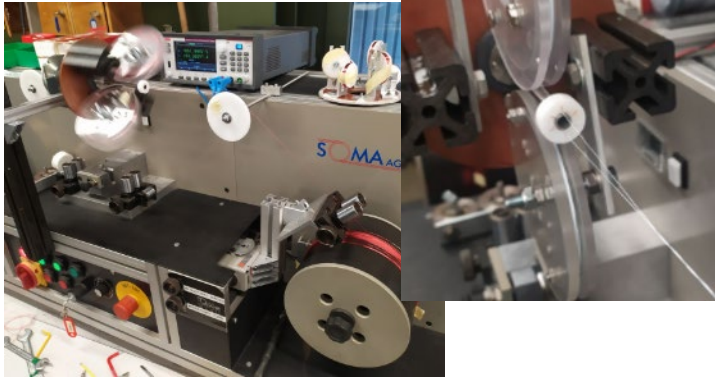
# Quench detection: instrumentation



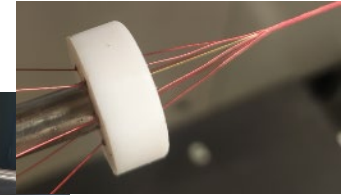
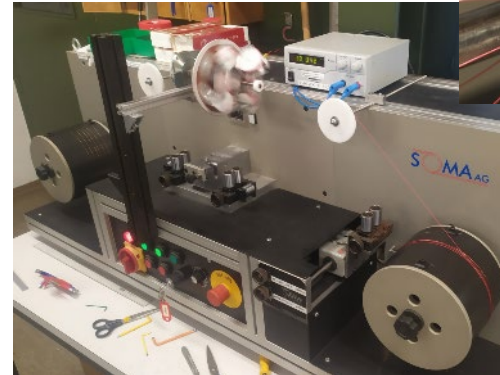
Quench detection embedded in G10 support plate:

- SQD twisted-pair (bronze-route Nb<sub>3</sub>Sn, OD 0.2 mm)
  - Shielded thermocouple chain (type K, OD 0.2 mm)
  - FBG optical fiber in Teflon capillary tube (ID 0.6 mm)...
- All three as non-invasive instruments, each ~10 m long

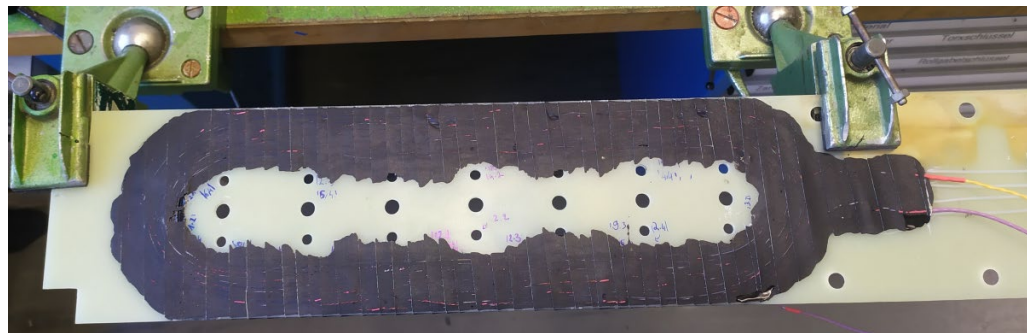
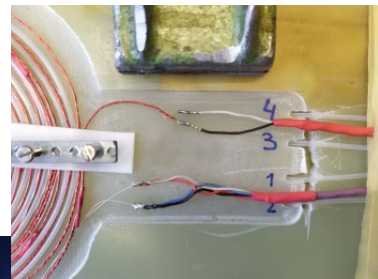
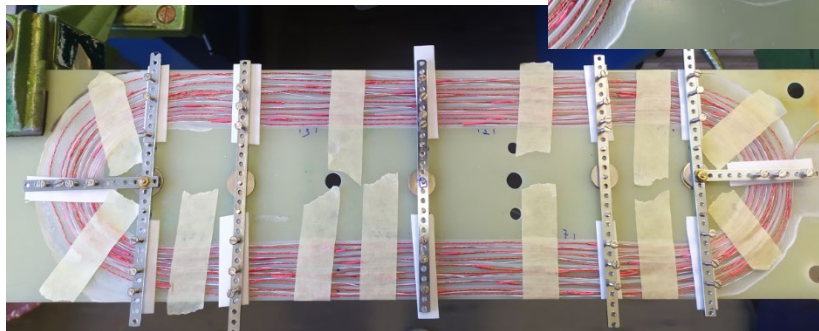
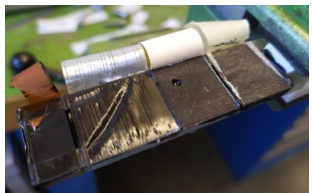
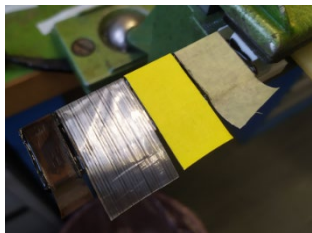
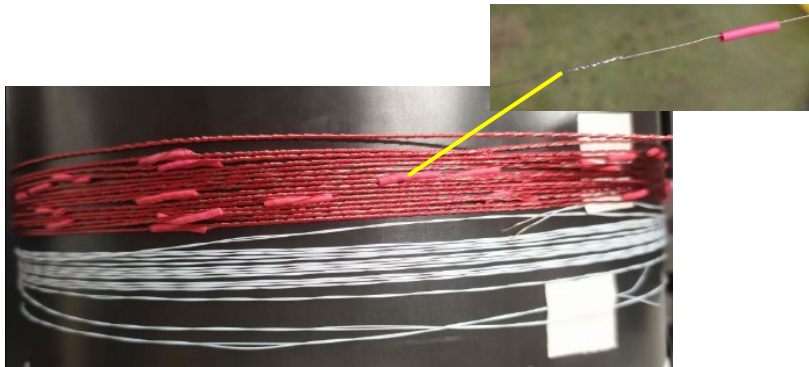
Twisted pair



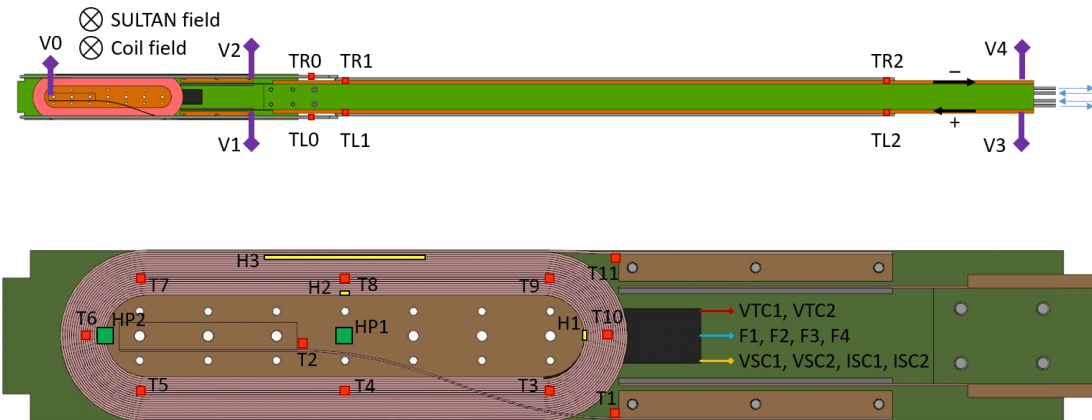
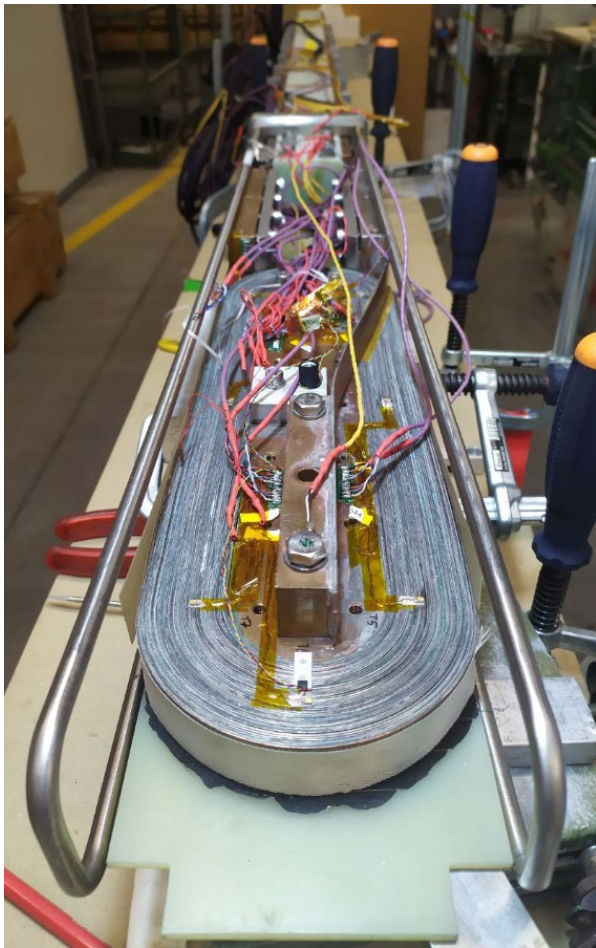
6-around-1 shield



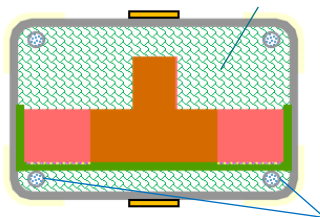
# Quench detection: assembly steps



# Current status



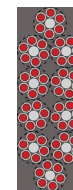
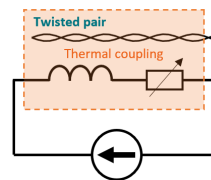
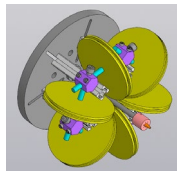
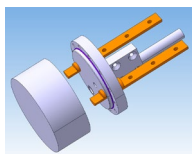
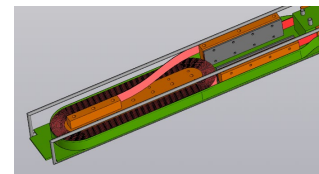
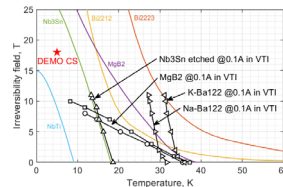
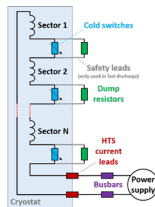
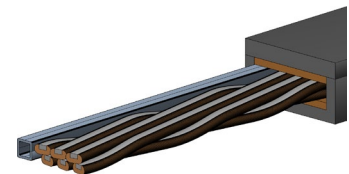
~10% DMSO (common cryoprotective agent)



Inlet cooling  
(supercritical helium)

→ Test in SULTAN  
in the coming weeks...

# THANK YOU FOR YOUR ATTENTION!



Former internship students at SPC-SG:

Max Bernheim, Alessio Rossi, Giuseppe Drago, Eduardo Baldo, Julia Haack

**There are lots of multi-disciplinary topics in applied superconductivity for fusion magnets, join the effort!**

e-mail: [nikolay.bykovskiy@psi.ch](mailto:nikolay.bykovskiy@psi.ch)